# **DU**Nuclear

#### **GPU Nuclear Corporation**

Post Office Box 388 Route 9 South Forked River, New Jersey 08731-0388 809 971-4000 Writer's Direct Dial Number:

> C321-92-2160 May 20, 1992

> > TE22

11

U.S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555

Dear Sir:

Subject:

Oyster Creek Nuclear Generating Station Docket No. 50-219 Licensee Event Report

This letter forwards one (1) copy of Licensee Evert Report 92-003.

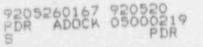
Sincerely,

John J. Barton Mice President and Director Oyster Creek

JJB\JJR Enclosure

cc: Administrator, Region 1 Senior NRC Resident Inspector Oyster Creek NRC Project Manager

(LER-COVLTRS)



GPU Nuclear Corporation is a subsidiary of General Public Utilities Corporation

i 	-							and the second second	A ALWANT CHINA	A SHARE A REPORT OF A DESCRIPTION OF A D	-	-		
NRC FORM 1848 3-631			LIC	ENSE	E EVEN	VT RE	PORT	(LER)				0445 840	2180-0104	
A CONTRACTOR OF A CONTRACTOR O						Replaced Article South		and the second	-			the sum of states	7201	
Oyster Cree	6k ····									I O I O I		1	1 OF	0.13
TITLE IAI	C. P.	na china ina manina managan					*****	a principal de la construcción de l	0.10	12121	W 16 1	in the second	Laurenterin	
Inadvertent M	anua1	Activati	on of an	t Ep	ineer		fety 1	Feature 4	Due t	o Pers	sonne	1 Err	'9 P	
IVENT DATE (8)		LER NUMBER	Spring sugarant submittee		IT DATE	(17)	Annual Control of Annual	statement of the local division of the local	and the second second	TIES INVOL		Contractions and		is an advertised to be
MONTHE DAY I YEAR	I VEAR	SEQUENTIAL NUMBER	NEVERONI NUMBER	NONTH	. 4.4	YEAR I		FABILITY NO	5. NO 8.8		DOCKET			
1.11.11.11.1								مديوما وقد وتورد ومد			0 151	0101		and min
04 20912	92	101013	010		2101	9121					0 . 6 .	0 1 0 1	0.1 1	1
OPERATING	THIS RE	PORT IS BURBAITTE	D PURBUANT T	0 THE RE	CLINES 3	NTE OF 1	CPR 6 /0	Suace and ar mare	e er the h	and the second	e an	and some de	construction with	o cana ficense
MODIE (E)	20.	402(6)		20.8051	d.			60.784412815H1	and a line some		22.2	71 de l	Contract of the output of the	
POWER LEVEL	providence of	406 IoH110		50.98 (a)				80.73411211VI				71 641		
101 11 1010	CARACTER	ACREAN 311 2100	-	60.36Lat	10.00		an and a	60.7961(211/6)				- 14 1814 191	eny is also (unt. NRC	
	processing.	408 to 111 1 (tr)		80.73iaH			and the second	86.734s1(2)1040		1				
	70.	1011111111	and excel	69.7361	5811885		a seat second	80.7356HET(a)						
and and a second to be in the second second second	Constant and a	and the second second second second			TOATHOT	FOR THIS	LER (12)	lange good target and make y				red r. and with some		
Paul Cerveni	ka i								1.0	A CODE I	ELEPHON	I NUMB	Rel Report to the lateral data of	
i and i series in										10,9	9.7	ti ai	4 8	9.4
	erorea dise viculei	COMPLETE	TINE LINE FOR	EACH CO	MPONENT	FAILURE	DERCRIES		and the second second	edu see drives see			en maderer oder	in the suffer strate and
CAUSE ISYSTEMI COM	PONENT	MANUFAC. TURER	TO HPRDS			CAUSE	SVETEN	COMPONENT		NUFAC-	NEPORT			
111	1.1	dista in							1.1					
	1.1	1.1.1				1	1.1	1.1.1		44				
Services and the strength of the service of the ser	end en order se ad	ELIPPL EME	INTAL REPORT	EXPECTE	0114		denne der and			EXPECTE		MONTH	DAY I	YEAN
e rest manual	h			-	-					SUBMISSIC	IN I			
ABSTRACT (L/mit TO 1400 a	and the second second	an development of the case opposite on the	VALUE AND DESCRIPTION OF TAXABLE	X	NO		-					mandanand	-	
approxim Service M operator procedury position valves for the Conta 825 gallo attribut involved The Play signific function acoustic electrom was star was held individu multiple	ately Water e to ed the or the ainme ed to proc ance. s wou ance. s wou and atic s ted at l, and at in actio	1992 at 100% powe System 1 forming th stop the e system n a Drywell nt Spray if Torus wa o operator edure ste ransient The real d be una thermocour relief val t 1730 how d appropr volved. on statema rade prog	r and Pr Pump Op he surv Contain ode swi Spray mo Pump wit iter ent r error p which Review sults o ffected uple mon ves. Thurs and iate pe Procedu ents con	ocedu erabi eilla ment tch to ode. hin 2 ered . A cont Grou f the by t itors ne PTF succe rscnr ral c	ure 60 ility nce ( Spray o the The o 29 sec the D cont ained ip (I rev: the ev s ass RG rec sesful hel a change id in	07.4.0 and overla AUTO perat conds rywel tribu seve PTRG) iew d vent coiat commen lly ce otion es ha the i	004, C In-ser boked posit or re- bor re- bor re- ting ting ting ting ting ting was eterm with t ed wi used to mplet s wer ve al nvolve	ontainme vice Tes a porti proceede lon. Th cognized ing this he cause cause to ction st convene ined tha the poss th the m esting o e taken so been ed proced	nt Sp t wa on c ad to is li the of t t this atemi t al ible ain f the 20 h with made lure	oray an s in p of a s o next nes up error iod ap his oc is eve ents. o det 1 safe except steam se sys ours. h resp to se step.	d Eme rogre tep the and s proxi curre nt wa ermin safe tems, A cr ect t epara An o	rgend ss.Th in th whice syste ecure matel mode as th elate of th ty ar whice itigute to th te th ngoin	cy he ch ed ls he he he he he he he he he	

LICENS	E EVENT REPORT (LER) TEXT	CONTINU	ATIO	N		UCLEAR REG 	HE NO. 116		
FACILITY HAME (1)	DOCKET NUMBER 12		Period and Address	LER NUM	BER 181		FAG	8. (31	Accession a
			~ 8 A.R	ARQUE NUE	DER	REVISION COMBER		-	
Oyster Creek	0 5 0 0 0	0   2  1  9	9 2		013	- 010	012	IF C	3
TEXT IN more assess at required, use additional NRC Form	JHEA'%/ (17)			-	Const in Constanting of	and the second second second	de la radició menorialme		

#### DATE OF OCCURRENCE

The event occurred on April 20, 1992 at approximately 1255 hours.

### IDENTIFICATION OF OCCURRENCE

While performing a surveillance test on the Containment Spray System, a portion of a step was overlooked which resulted in an inadvertent manual actuation of an engineered safety feature. This ovent is considered reportable in accordance with 10CFR 50.73.(a).(2).(iv)

### CONDITIONS PRIOR TO OCCURRENCE

The Containment Spray (EIIS-BP) and Emergency Service Water (EIIS-BS) System 1 Pump Operability and Inservice Test procedure was in progress. The reactor was operating at approximately 100% power.

#### DESCRIPTION OF OCCURRENCE

On April 20. 1992 at approximately 1255 hours, Procedure 607.4.004, Containment Spray and Emergency Service Water System 1 Pump Operability and In-service Test was in progress. The Containment Spray System valves were lined up in the test mode with Containment Spray pump 518 operating. In this mode water from the Torus is pumped through the Containment Spray heat exchangers and then returned to the Torus via a test loop. The operator performing the surveillance overlooked a portion of a step in the procedure to stop the Containment Spray pump and proceeded to the next step which positioned the system mode switch to the AUTO position. The AUTO position lines up the system valves (CFI-ISV) for the Drywell Spray mode. The operator recognized the error and secured the Containment Spray Pump within 29 seconds. During this period approximately 825 gallons of Torus water entered the Drywell. Drywell pressure prior to the event was 1.19 psi. Drywell pressure initially decreased by .15 psi due to the cooling effects of the spray and then increased to a peak of '.4 psi which was only .2 psi above the initial drywell pressure. Drywell bulk temperature decreased by approximately six degrees. The Drywell Sump High Leakrate Alarm (EIIS-IJ) was received and cleared approximately five times during the .ext 40 minutes Sue to the event. The plant continued to operate at approximately 100% power during and after the event.

#### APPARENT CAUSE OF OCCURRENCE

The cause of this occurrence is attributed to operator error. The operator overlooked an action statement to stop the Containment Spray Pump contained within a step in the procedure and also failed to utilize self checking methods before performing the next critical step. A contributing cause to this event was the involved procedure step which contained several action statements. LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

5. NUCLEAR REQULATORY COMMINERON

TREAVED ONE NO 1150-0104 IVERES EDITES

ACILITY NAME IT	CODKET NUMBER (2)	LER NUMBER (6)	FAGE (3)	
		「主来来」「「主はい本句で」は、「「主い情報」」		
Oyster Creek		0 912 mil 0 10 1 3 mil 0 0 0	nit hen	

HRC'PHIM MAA

## ANALYSIS OF OCCURRENCE AND SAFETY ASSESSMENT

The Plant Transient Review Group (PTRC) was convened to determine the significance of this event. A review of plant data indicated there were no immediate or obvious adverse effects on any equipment contained in the drywell. A listing of safety related equipment contained in the Drywell was reviewed with respect to the Environmental Qualification and failure mode of the equipment to determine if any safety related function was in question.

The results of the review determined that all safety related functions would be unaffected by the event with the possible exception of the acoustic and thermocouple monitors associated with the main steam safety and electromatic relief valves. The PTRG recommended testing of these systems, which was started at 1730 hours and successfully completed at 2320 hours.

Based upon the above safety significance of this event is considered minimal.

#### CORRECTIVE ACTION

A critique was held and appropriate personnel action was taken with respect to the individual involved in this event.

Procedural changes were made in the referenced procedure to separate to multiple action statements contained in the involved procedure step. An ongoing procedure upgrade program includes a review of procedures for multiple action statements.

Management discussions with the operators involved concerning the need to perform Self-Checking, and the Work Performance Standard on Procedure Compliance were held. Management determined the appropriate training/requalification of the operator performing the surveillance, prior to resuming licensed duties.

The expectations of Operations Management regarding compliance with the Operations Department Standard on Procedure Compliance have been communicated to all Operations Department Personnel.

A critique of this event was issued as required reading for all Licensed/Non-Licensed Operations personnel and all staff License or Certification holders.

Development of the concept of Crew Self-Checking, including a training module for presentation to each of the operating crews will be considered for implementation.

Evaluation of the need for and, where necessary, refresher self-checking training will be provided for all Licensed/Non-Licensed Operations Department personnel.

### SIMILAR EVENTS

None.