

Janaury 23, 1996

Mr. Robert E. Busch
President - Energy Resources Group
Northeast Utilities Service Company
c/o Mr. Richard M. Kacich
P.O. Box 128
Waterford, CT 06385

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - MILLSTONE NUCLEAR POWER STATION, UNIT 2 (TAC NO. M93955)

Dear Mr. Busch:

The NRC staff has reviewed your request of October 24, 1995, for an amendment that would defer the inspection of the steam generator tubes until the next refueling outage or no later than October 20, 1997. The plant TS currently require an inspection of the steam generator tubes no later than October 20, 1996. The proposed amendment is a one-time extension of the inspection interval.

Additional information is required in order for the staff to complete its review. The enclosed Request for Additional Information identifies the information that is required. Please provide your response within 45 days from the receipt of this request. This requirement affects nine or fewer respondents, and therefore, is not subject to the Office of Management and Budget review under P.L. 96-511.

Sincerely,

Original signed by:

Guy S. Vissing, Senior Project Manager
Northeast Utilities Project Directorate
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-336

Enclosure: Request for Additional Information

cc w/encl: see next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

January 23, 1996

Mr. Robert E. Busch
President - Energy Resources Group
Northeast Utilities Service Company
c/o Mr. Richard M. Kacich
P.O. Box 128
Waterford, CT 06385

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Sincerely,

A handwritten signature in cursive script, appearing to read "Guy S. Vissing".

Guy S. Vissing, Senior Project Manager
Northeast Utilities Project Directorate
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-336

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R. Busch
Northeast Utilities Service Company

Millstone Nuclear Power StationR
Unit 2

cc:

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REQUEST FOR ADDITIONAL INFORMATION
CONCERNING
REQUEST TO DEFER THE INSPECTION OF STEAM GENERATOR TUBES
FOR
MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2
DOCKET NO. 50-336

QUESTIONS:

1. What was the total length of time following the steam generator replacement up to the first tube inspections that the plant was in Modes 1, 2 or 3? What is the total time projected up through October 20, 1997?
2. A partial inspection of the steam generators was completed following Cycle 12 operation. Reference 1 indicated that the inspection scope was limited to 30-percent of the tubes. Provide the following additional information regarding the scope of these inspections:
 - Number of tubes inspected within each steam generator
 - Tube locations inspected with reference to type of probes used (e.g., full length bobbin, tubesheet RPC)
3. Provide a list of indications identified by eddy current, their location and the postulated degradation mechanism for the results of the steam generator tube inspections following Cycle 12 operation.
4. Reference 1 describes an eddy current indication identified in Steam Generator 1 (R140/C79) with a depth determined at approximately 22-percent through wall. Based on the inspection results, was this indication attributed to a corrosive mechanism or to mechanically-induced damage (e.g., wear)? The text also states that the indication was confirmed in a rotating probe inspection. If the damage mechanism was believed to be stress corrosion cracking, state the basis for allowing the tube to remain in service considering that a rotating pancake coil probe has a detection threshold on the order of 40-percent for such indications.
5. Page 4 in Reference 1 states that "only 27 [tubes] has sludge indications." Does this statement refer to tube indications (degradation) or to indications from the eddy current signal that a sludge pile exists surrounding each of these 27 tubes?
6. Provide a table listing the monthly average values of the steam generator blowdown control parameters referenced in EPRI "PWR Secondary Water Chemistry Guidelines" for power operation since the start of Cycle 12. Also provide the length of time that significant out-of-specification secondary water chemistry conditions (e.g., Action Levels 2 and 3) existed in the replacement steam generators.

Reference 1 - Attachment 1 to "Millstone Nuclear Power Station, Unit No. 2, Proposed Revision to Technical Specifications, Steam Generator Surveillance Requirement Extension," October 24, 1995.

Enclosure