NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Washington Nuclear Plant - Unit 2	0 5 0 0 0 3 9 7	1 OF 0 2

TITLE (4)
RCIC Isolations on High Steam Flow

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 8	0 2	8 4								0 5 0 0 0
0 8	2 3	8 4	8 4	- 0 8 2 -	0 0	0 8	3 0	8 4		0 5 0 0 0

OPERATING MODE (9)	5	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 0	20.402(b)	20.405(c)	[X] 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	[X] OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	50.72(b)(2)(ii)
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
R. L. Koenigs, Compliance Engineer	AREA CODE 5 0 9 3 7 7 - 2 5 0 1 Ext. 2279

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	B N	P D I S	B 0 8 0	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
[X] YES (If yes, complete EXPECTED SUBMISSION DATE)	[ ] NO		0 1	3 0	8 5

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 8/2/84 and 8/23/84 the reactor was shutdown with the Reactor Core Isolation Cooling (RCIC) System in use to maintain reactor water level. On both dates spurious high steam flow isolations occurred. The isolations occurred at reactor pressures of between 150 to 300 psig.

It was verified that no steam leakage had occurred. After the 8/2/84 event the Condensate System was placed into operation to provide reactor water makeup. Following the 8/23/84 event the RCIC was returned to service.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Washington Nuclear Plant - Unit 2	0 5 0 0 0 3 9 7	8 4	— 0 0 2	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Plant Conditions

	8/2/84	8/23/84
a) Power Level -	0%	0%
b) Operational Mode -	3*	3

*Mode switch in refueling position to support SRM surveillance.

Event

On 8/2/84 and 8/23/84, following reactor shutdowns (all control rods fully inserted) Reactor Core Isolation Cooling (RCIC) System isolations occurred while the RCIC system was being used to maintain reactor water level. The isolations resulted from Division II RCIC high steam flow signals, and occurred at reactor pressures of between 150-300 psig during reactor cooldown.

In both instances no other indications were present to support actual high steam flow conditions.

Immediate Corrective Action

Plant personnel inspected the piping for leaks and found none. Following the 8/2/84 event the Condensate System was placed in operation and used for reactor water level control. The reference leg for RCIC-DPIS-13B (instrument which originated signal) was filled with water and the instrument recalibrated.

After the 8/23/84 event the system was reinspected for leaks and the RCIC System returned to service.

Further Corrective Action

An investigation to determine the cause of these spurious trips has been initiated. An additional pressure transmitter will be placed in line with the Differential Pressure Indicating Switch for RCIC Div. 2 Isolation. This additional pressure transmitter will be connected to the Transient Data Analysis System for continuous recording. The Supply System will provide a follow-up to this LER with the results of the above investigation and any corrective actions which are identified.

Safety Significance

In both cases the Reactor had been shutdown (all control rods were full in) and all other safety systems were operational and in standby. In both cases there was no hazard to the safety of the Plant or that of the public.

Washington Public Power Supply System
P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

August 30, 1984

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 84-082

Dear Sir:

Transmitted herewith is Licensee Event Report No. 84-082 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

This is the follow-up report to the verbal notification given at 0100 hours on August 2, 1984 and at 0205 August 23, 1984.

Very truly yours,

C M Powers for
J. D. Martin (M/D 927M)
WNP-2 Plant Manager

JDM:mm

Enclosure:
Licensee Event Report No. 84-082

cc: Mr. John B. Martin, NRC - Region V
Mr. A. D. Toth, NRC - Site (901A)
Ms. Dottie Sherman, ANI
INPO Records Center - Atlanta, GA

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