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NRC Form 386 (9-83)

NAC Form 384A (9-83) LICEN	SEE EVENT REPORT (LER) TEXT CONTI	NUATIO	UL	U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMS NO. 3150-0104 EXPIRES: 8/31/85				
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TEXT (If more space is required, use additional NRC Form 386A's) (17)

On August 4, 1984, at 1355, a turbine (TRB;TA) trip occurred, resulting in an automatic reactor (RCT;AB) trip due to loss of load. The Plant was operating at approximately 48% power at the time of the occurrence. No threat to public health or safety resulted.

The turbine trip was attributed to a loss of electro hydraulic control (EHC) fluid pressure, which allowed all major turbine (V;TA) operation values to close. Subsequent investigation determined that a fitting on the discharge line from EHC Pump P-19A (P;TG) had backed completely off, resulting in a loss of EHC fluid inventory, as the EHC fluid was pumped directly out of the system through the open discharge line. The fitting had worked loose as the result of excessive system vibration. A bracket which provides rigid support to the discharge line was noted to be missing. The bracket had been removed during the 1983-1984 refueling outage and was inadvertently not replaced during system reassembly. The root cause of the incident, therefore, was the failure to reinstall the support bracket on the EHC discharge line. The bracket was subsequently reattached in the appropriate location. The occurrence and its significance will be reviewed with management personnel from the responsible group.

During the incident, the Reactor Protection System functioned as designed to automatically shut down the reactor. The magnitude of the transient resulting from a reactor trip varies with power level. A reactor trip from full power, however, is an analyzed occurrence which would not place the Plant in a condition which is outside of its design basis. Additionally, Safeguards Bus 1-C (BU;EB) did not fast transfer to start-up power, but was picked up by Emergency Diesel Generator 1-1 (DG;EK). Investigation revealed a blown fuse (FU;EKO in the undervoltage feature of Bus 1-C Supply Breaker 152-106 (BRK;EB). The fuse was subsequently replaced, and Bus 1-C was transferred to start-up power.



General Offices: 1945 West Parnall Road, Jackson, MI 49201 • (517) 788-0550

August 31, 1984

US Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

DOCKET 50-255 - LICENSE DPR-20 -PALISADES PLANT - LICENSEE EVENT REPORT 84-015 (REACTOR TRIP DUE TO LOSS OF ELECTRO HYDRAULIC FLUID PRESSURE)

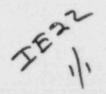
Attached please find Licensee Event Report 84-015 (Reactor Trip Due to Loss of Electro Hydraulic Fluid Pressure) which is reportable to the NRC per 10 CFR 50.73(a)(2)(iv).

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Ralph R Frisch Senior Licensing Analyst

CC Administrator, Region III, USNRC Director, Office of Nuclear Reactor Regulation NRC Resident Inspector - Palisades

Attachment



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