

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Palisades Nuclear Plant	DOCKET NUMBER (2) 0 5 0 0 0 2 5 5	PAGE (3) 1 OF 0 2
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TITLE (4)
Reactor Trip Due to Loss of Electro Hydraulic Fluid Pressure

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)					
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					
0	8	0 4 8 4	8 4	0 1 5	0 0	0 8 3 1 8 4	NA			DOCKET NUMBER(S) 0 5 0 0 0				
									NA			0 5 0 0 0		

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)

OPERATING MODE (9) N	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(a)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
	<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.39(a)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(a)
	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.39(a)(2)	<input type="checkbox"/> 50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 308A)
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(vii)(A)	
	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(vii)(B)	
	<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME David W Rogers; Technical Engineer; Palisades	TELEPHONE NUMBER AREA CODE: 6 1 6 7 6 4 - 8 9 1 3
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	E	B B K R	A 1 8 0	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On August 4, 1984, loss of electro hydraulic control (EHC) fluid pressure resulted in a turbine trip, an an automatic reactor trip. The Reactor Protection System (RPS) functioned as designed to shut down the reactor. No threat to public health or safety resulted.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On August 4, 1984, at 1355, a turbine (TRB;TA) trip occurred, resulting in an automatic reactor (RCT;AB) trip due to loss of load. The Plant was operating at approximately 48% power at the time of the occurrence. No threat to public health or safety resulted.

The turbine trip was attributed to a loss of electro hydraulic control (EHC) fluid pressure, which allowed all major turbine (V;TA) operation valves to close. Subsequent investigation determined that a fitting on the discharge line from EHC Pump P-19A (P;TG) had backed completely off, resulting in a loss of EHC fluid inventory, as the EHC fluid was pumped directly out of the system through the open discharge line. The fitting had worked loose as the result of excessive system vibration. A bracket which provides rigid support to the discharge line was noted to be missing. The bracket had been removed during the 1983-1984 refueling outage and was inadvertently not replaced during system reassembly. The root cause of the incident, therefore, was the failure to reinstall the support bracket on the EHC discharge line. The bracket was subsequently reattached in the appropriate location. The occurrence and its significance will be reviewed with management personnel from the responsible group.

During the incident, the Reactor Protection System functioned as designed to automatically shut down the reactor. The magnitude of the transient resulting from a reactor trip varies with power level. A reactor trip from full power, however, is an analyzed occurrence which would not place the Plant in a condition which is outside of its design basis. Additionally, Safeguards Bus 1-C (BU;EB) did not fast transfer to start-up power, but was picked up by Emergency Diesel Generator 1-1 (DG;EK). Investigation revealed a blown fuse (FU;EKO in the undervoltage feature of Bus 1-C Supply Breaker 152-106 (BRK;EB). The fuse was subsequently replaced, and Bus 1-C was transferred to start-up power.



Consumers
Power
Company

General Offices: 1945 West Parnall Road, Jackson, MI 49201 • (517) 788-0550

August 31, 1984

US Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

DOCKET 50-255 - LICENSE DPR-20 -
PALISADES PLANT - LICENSEE EVENT REPORT 84-015 (REACTOR TRIP DUE TO LOSS OF
ELECTRO HYDRAULIC FLUID PRESSURE)

Attached please find Licensee Event Report 84-015 (Reactor Trip Due to Loss
of Electro Hydraulic Fluid Pressure) which is reportable to the NRC per 10 CFR
50.73(a)(2)(iv).

Ralph R Frisch

Ralph R Frisch
Senior Licensing Analyst

CC Administrator, Region III, USNRC
Director, Office of Nuclear Reactor Regulation
NRC Resident Inspector - Palisades

Attachment

OC0884-0017-NL02

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