NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
LaSalle County Station Unit 2	0 5 0 0 0 3 7 4	1 OF 0 3

TITLE (4)
Reactor Water Clean Up High Differential Flow Isolation

EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
											N/A	0 5 0 0 0
0 8	0 1	8 4	8 4	—	0 4 4	—	0 0	0 8	1 7	8 4	N/A	0 5 0 0 0

OPERATING MODE (9)	2	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)					
POWER LEVEL (10)	0 0 1	20.402(b)	20.405(c)	X	50.73(a)(2)(iv)		73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)		50.73(a)(2)(v)		73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		
		20.405(a)(1)(iv)	50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		
		20.405(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
JoAnn M. Shields, Extension 330	AREA CODE 8 1 5 3 5 7 - 1 6 7 6 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	C E	I R V	L 2 6 5	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X	YES (If yes, complete EXPECTED SUBMISSION DATE)		1 1	3 1	8 4
	NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On August 1, 1984, at 1451, with Unit 2 in Start Up, Reactor Water Clean Up isolated on high differential flow. At the time of the event, vessel level was being controlled by RWCU Blowdown Flow. A lifted relief valve, combined with the reactor start up, caused the isolation. Safe plant conditions were maintained at all times. A work request was written to investigate the problem with the lifting relief valve.

IE22
1/1

NRC Form 366
(9-83)

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
LaSalle County Station Unit 2	0 5 0 0 0 3 7 4	8 4	— 0 1 4	— 0 0	0 2 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

I. EVENT DESCRIPTION

On August 1, 1984 at 1451, with Unit 2 in Start Up Mode and at about 800 psig, Reactor Water Clean Up (CE, RWCU) isolated on high differential flow (JM). At the time of the event, vessel level was being controlled by RWCU blowdown flow. While running at 800 psig, the "B" regenerative heat exchanger shell side relief valve, 2G33-F340B, lifted, venting to the reactor building equipment drain tank (WD). The system isolated according to design upon reaching the 70 gpm isolation setpoint. Safe plant conditions were maintained at all times.

II. CAUSE

The valve, 2G33-F340B, RWCU "B" regenerative heat exchanger shell side relief valve, lifted, venting water to the reactor building equipment drain tank. The loss of flow through the RWCU design flowpaths added to the differential flow value and isolated the system.

To compound the event, the RWCU differential flow instruments are designed for operation at rated pressures and temperatures. As a result, the various flow loops are calibrated at the appropriate water density expected during steady state operation. In start-up, however, the water densities will not be at rated conditions, causing the instruments to indicate conservatively.

The above two elements, a lifted relief valve and a reactor start-up, combined to give the isolation.

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

The event was of minimal significance as the system operated according to design. Flow out the relief valve was routed to the reactor building equipment drain tank. Safe plant conditions were maintained at all times.

IV. CORRECTIVE ACTION

A work request was written to investigate the relief valve lifting at less than design pressure. Resolution of this work will be tracked by AIR 01-84-67065.

V. PREVIOUS OCCURRENCES

Other reports of relief valves lifting at less than design pressure are detailed in LER 374/84-13-00 and LER 374/84-23-00.

Other reports of isolations while the reactor is in start up and blowing down to the condenser have occurred on Unit 1 and Unit 2 and are detailed in LER's 373/84-030-00, 84-033-00, 84-040-00 and 374/84-029-00, 84-041-00.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)				PAGE (3)
		YEAR		SEQUENTIAL NUMBER	REVISION NUMBER	
LaSalle County Station Unit 2	0 5 0 0 0 3 7 4	8 4	—	0 4 4	— 0 0	0 3 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

VI. NAME AND TELEPHONE NUMBER OF PREPARER

JoAnn M. Shields, (815)357-6761, Extension 330.

Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

August 17, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #84-044-00, Docket #050-374 is being submitted to your office in accordance with 10 CFR 50.73.

G. J. Diederich
Superintendent
LaSalle County Station

GJD/MLD/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

IE22
1/1