

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) CRYSTAL RIVER UNIT 3	DOCKET NUMBER (2) 0 5 0 0 0 3 0 2	PAGE (3) 1 OF 03
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TITLE (4)
Failure to Perform Required Radioactive Gaseous Effluent Sample

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
07	30	84	84	017	00	08	29	84	N/A		0 5 0 0 0
									N/A		0 5 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

OPERATING MODE (8) 1	20.402(b)	20.406(e)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0.97	20.406(a)(1)(i)	50.38(e)(1)	50.73(a)(2)(v)	73.71(a)
	20.406(a)(1)(ii)	50.38(e)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in-Text, NRC Form 366A)
20.406(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)		
20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)		
20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)		

LICENSEE CONTACT FOR THIS LER (12)

NAME W. K. Bandhauer, Nuclear Safety Supervisor	TELEPHONE NUMBER
	AREA CODE: 9 0 4 7 9 5 - 6 4 8 6

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC TURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFAC TURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (if yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH DAY YEAR
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During a routine review of Surveillance Procedures, it was discovered that a sample analysis required by Technical Specifications was not performed within the required period. The sample analysis, an Auxiliary Building and Fuel Handling Area Exhaust Duct Monitor, RM-A2 (IL), Grab Sample analysis, is required to be performed between two and six hours following a change in power level exceeding 15% of rated thermal power within one hour. The power level change occurred at 1354 on July 30, 1984, with the missed sample noted at 1235 on July 31, 1984. A sample was immediately analyzed (1245 on July 31, 1984), resulting in a time delay of nearly 23 hours following the power level change. Study of the applicable alarm printout and chart recorder revealed no significant fluctuations in the RM-A2 countrate during the period of interest. Applicable Surveillance Procedures will be changed to highlight Technical Specification sample requirements.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		8 4	- 0 1 7	- 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 368A's) (17)

EVENT DESCRIPTION

During a routine review of Surveillance Procedure SP-735, "Gaseous Radwaste Release Surveillance Program-Auxiliary Building-Continuous Mode", it was discovered that a gas grab sample analysis required by Section 4.11.2.1.2 of Technical Specifications was not performed. The sample of concern is an Auxiliary Building and Fuel Handling Area Exhaust Duct Monitor, RM-A2, Grab Sample analyzed for Principal Gamma Emitters in accordance with Item C of Table 4.11-2 of Technical Specifications. This sample analysis is required to be performed between two and six hours following a change in power level exceeding 15% of rated thermal power within one hour. Technical Specification Section 11, "Radioactive Effluents", which includes Section 4.11.2.1 and Table 4.11-2, was added in Revision 69, effective July 1, 1984; previous revisions of Technical Specifications did not contain these requirements.

At 1354 on July 30, 1984, Crystal River Unit 3 completed a change in power level exceeding 15% within one hour. The Chemistry Section was notified of the power change per step 4.4.2 of Operating Procedure OP-204, "Power Operation"; however, Chemistry Section personnel failed to perform the required sample analysis because of unfamiliarity with the requirement. Surveillance Procedure SP-735 listed the analysis as a required item on a Surveillance Table, but did not cross-reference required samples to applicable plant conditions or transients. In addition, Surveillance Procedure SP-442, "Special Conditions Surveillance Plan", referenced only SP-709, "Reactor Coolant and Decay Heat Non-Scheduled Surveillance", in the section governing requirements following thermal power change of greater than 15% within one hour and did not reference SP-735.

When the failure to obtain the sample was discovered at 1235 on July 31, 1984, a sample was obtained and analyzed immediately (1245, July 31, 1984). The results were normal.

SAFETY CONSIDERATIONS

This event had no impact on the health and safety of the public. Inspection of the Environmental Radiation Monitor Recorder, RM-R6 (IL), showed that the countrate on RM-A2 did not change significantly before, during, or after the power change, indicating no change in the dose rate to the public from the effluent stream monitored by RM-A2. The alarm printout for the period from 1354 on July 30, 1984 to 1245 on July 31, 1984 was examined for RM-A2 Warning or High Alarms/Trips. The Warning Alarm Setpoint is three times background, while the High Alarm/Trip Setpoint is 10% of the Technical Specification limit. No alarms on RM-A2 were noted for the period of interest, indicating no significant change in the dose rate to the public. In addition, the results of the Reactor Coolant sample analysis taken as required by SP-709, "Reactor Coolant and Decay Heat Non-Scheduled Surveillance", showed no significant change in the Reactor Coolant iodine concentration, indicating no change in the source term, further indicating no significant change in the dose rate to the public.

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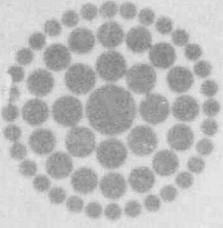
TEXT (If more space is required, use additional NRC Form 366A's) (17)

CORRECTIVE ACTION

To preclude recurrence of this and similar events, applicable Surveillance Procedures are being changed to highlight Technical Specification sample requirements. Surveillance Procedure SP-442, "Special Conditions Surveillance Plan", will be revised to add a reference to Surveillance Procedures SP-731, "Gaseous Radwaste Release Surveillance Program-Reactor Building-Continuous Mode", and SP-735, "Gaseous Radwaste Release Surveillance Program-Auxiliary Building-Continuous Mode" in the section governing requirements, following thermal power changes of greater than 15% in one hour. Applicable Chemistry-related Surveillance Procedures will be revised to incorporate an additional "Chemrad Report" solely for analysis of samples required following startup, shutdown, or power change of greater than 15% in one hour.

PREVIOUS SIMILAR EVENTS

This is the first event concerning failure to perform an Auxiliary Building exhaust gas sample analysis within the specified time interval following a change in power level exceeding 15% of rated thermal power within an hour.



**Florida
Power**
CORPORATION

August 29, 1984
3F0884-18

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Crystal River Unit 3
Docket No. 50-302
Operating License No. DPR-72
Licensee Event Report No. 84-017-00

Dear Sir:

Enclosed is Licensee Event Report (LER) No. 84-017-00 which is submitted in accordance with 10 CFR 50.73.

Should there be any questions, please contact this office.

Sincerely,

G. R. Westafer
Manager, Nuclear Operations
Licensing and Fuel Management

RHT/feb

Enclosure

cc: Mr. James P. O'Reilly
Regional Administrator, Region II
Office of Inspection & Enforcement
U.S. Nuclear Regulatory Commission
101 Marietta Street N.W., Suite 2900
Atlanta, GA 30323

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