



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 167
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated July 13, 1990, as supplemented by letters dated November 27, 1991, and February 21, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 167, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Charles L. Miller

Charles L. Miller, Director
Project Directorate I-2
Division of Reactor Projects - I/11

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 7, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 167

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

<u>Remove</u>	<u>Insert</u>
247	247
248	248
252a	252a
253	253
254	254
260	260

PBAPS

Meeting Frequency

- 6.5.1.4 The PORC shall meet at least once per calendar month and as convened by the PORC Chairman or his designated alternate(s).

Quorum

- 5.1.5 A quorum of the PORC necessary for the performance of the PORC responsibilities and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate(s) and four members or their alternates.

Responsibilities

- 6.5.1.6 The Plant Operations Review Committee shall be responsible for:
- a. Review of 1) Administrative procedures and changes thereto, 2) new procedures requiring a 10 CFR 50.59 safety evaluation, and 3) proposed changes to procedures requiring a 10 CFR 50.59 safety evaluation.
 - b. Review of all proposed tests and experiments that affect nuclear safety.
 - c. Review of all proposed changes to the Technical Specifications.
 - d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
 - e. Investigation of all violations of the Technical Specifications and shall prepare and forward a report covering evaluation and recommendations to prevent recurrence to the Plant Manager, the Vice President, Peach Bottom Atomic Power Station, and the Nuclear Review Board.
 - f. Review of facility operations to detect potential safety hazards.
 - g. Performance of special reviews and investigations and reports thereon as requested by the Chairman of the Nuclear Review Board.
 - h. Review of all reportable events required by 10 CFR 50.73.

PBAPS

6.5.1.6 Continued

- i. Review of the Plant Security Plan, and shall submit recommended changes to the Plan to the Plant Manager and the Nuclear Review Board.
- j. Review of the Emergency Plan, and shall submit recommended changes to the Plan to the Plant Manager and the Nuclear Review Board.
- k. Review of every unplanned release reportable under 10CFR 50.72 and 50.73, of radioactive material to the environs; evaluate the event; specify remedial action to prevent recurrence; and document the event description, evaluation, and corrective action and the disposition of the corrective action in the plant records.

Authority

6.5.1.7 The Plant Operations Review Committee shall:

- a. Recommend in writing, to the Plant Manager, for his approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(b) through (e) above constitutes an unreviewed safety question, as defined in 10 CFR 50.59.
- c. Provide immediate written notification to the Vice President, Peach Bottom Atomic Power Station, or in his absence, the Senior Vice President - Nuclear and the Nuclear Review Board of disagreement between the PORC and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Senior Vice President-Nuclear, and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.5.3 Procedure Review and Approval

- 6.5.3.1 Each new procedure and procedure change shall be reviewed by a Station Qualified Reviewer (SQR) and then by a Superintendent designated as responsible for the procedure. Administrative procedures (and changes thereto) shall be reviewed by PORC prior to approval by the Plant Manager or his designated alternate in accordance with Specification 6.1.1. The SQR shall not be the individual who prepared the new procedure or procedure change, he may however be from the same organization as the preparer. In addition to performing procedure reviews, the responsible Superintendents shall ensure that a sufficient complement of SQR's for their functional area is maintained. PORC shall have the option to review new procedures or procedure changes without the prior review and approval of the SQR and responsible Superintendent. Procedures reviewed in this manner will be approved by the Plant Manager.
- 6.5.3.2 Unless preempted by PORC as described in 6.5.3.1, the SQR shall review each new procedure or procedure change. The SQR shall render a determination in writing of whether or not a cross-disciplinary review of the new procedure or procedure change is necessary. If such a review is determined to be necessary, it shall be performed by the appropriate personnel. Upon completion of the SQR review, each new procedure or procedure change shall be reviewed by the Superintendent designated by Administrative procedures as the responsible Superintendent for that procedure. The Superintendent shall, as part of his review, decide whether or not a 10 CFR 50.59 Safety Evaluation is required. If a Safety Evaluation is not required, the new procedure or procedure change shall be approved by the responsible Superintendent or the Plant Manager prior to implementation.
- 6.5.3.3 If the responsible Superintendent determines that a new procedure or procedure change requires a 10 CFR 50.59 Safety Evaluation, the responsible Superintendent shall render a determination in writing of whether or not the new procedure or procedure change involves an Unreviewed Safety Question (USQ). The responsible Superintendent shall then forward the new procedure or procedure change with the associated Safety Evaluation to PORC for review. If an USQ is involved, prior NRC approval is required per 10 CFR 50.59 before implementation of the new procedure or procedure change.
- 6.5.3.4 Personnel recommended to perform the function of SQR shall be approved and designated as such by the PORC Chairman. The SQR shall be an individual from the responsible organization and knowledgeable in the functional area affected. The individual shall meet the qualifications of ANSI/ANS-3.1-1981, Selection, Qualification and Training of Personnel for Nuclear Power Plants, Section 4, excluding subsections 4.3.2 and 4.5. Individuals who do not possess the formal educational requirements specified shall not be automatically eliminated where other factors provide sufficient demonstration of their abilities. These factors, as listed in subsection 4.1 of ANSI/ANS-3.1, shall be evaluated on a case-by-case basis and approved and documented by the PORC Chairman.
- 6.5.3.5 Temporary procedure changes shall be reviewed and approved in accordance with Specification 6.8.3.
- 6.5.3.6 Records documenting the activities performed under Specification 6.5.3.1 through 6.5.3.5 shall be maintained in accordance with Specification 6.10.

PBAPS

6.6 Reportable Event Action

6.6.1 The following actions shall be taken for Reportable Events:

- a. The Commission shall be notified pursuant to the requirements of Section 50.73 to 10 CFR 50.
- b. Each Reportable Event Report submitted to the Commission shall be reviewed by the PORC and submitted to the NRB and the Vice President, Peach Bottom Atomic Power Station.

6.7 Safety Limit Violation

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Vice President, Peach Bottom Atomic Power Station, Plant Manager, and the NRB shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NRB and the Vice President, Peach Bottom Atomic Power Station within 10 working days of the violation.

6.8 Procedures

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained as follows, except as provided in Technical Specifications 6.8.2 and 6.8.3:

- a. Procedures that meet the requirements of Sections 5.1 and 5.3 of ANSI N18.7-1972
- b. Procedures that meet the requirements of Appendix "A" of USAEC Regulatory Guide 1.33 (November 1972)
- c. Procedures covering radiological effluent technical specification activities, including the Offsite Dose Calculation Manual and Quality Assurance Program for environmental monitoring using the guidance in Regulatory Guide 4.1, Revision 1, April 1975.

- 6.8.2 Each procedure and administrative policy of Technical Specification 6.8.1 and changes thereto, and any other procedure or procedure change that the Plant Manager determines to affect nuclear safety shall be reviewed and approved in accordance with Technical Specification 6.5.3 (and 6.5.1.6.a and 6.5.1.7.a, if required) prior to implementation. Each procedure of Technical Specification 6.8.1 shall also be reviewed periodically as set forth in Administrative procedures.
- 6.8.3 Temporary changes to procedures of Technical Specification 6.8.1 above may be made, provided:
- a. The intent of the original procedure is not altered.
 - b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
 - c. Within 14 days of implementation, the change is documented, reviewed by an SQR in accordance with Technical Specification 6.5.3.1, and approved by either the Plant Manager (or his designated alternate in accordance with Technical Specification 6.1.1) or the Superintendent designated by Administrative procedures as the responsible Superintendent for that procedure.
- 6.8.4 (Relocated into 6.8.1.c)

6.9 Reporting Requirements

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted to the NRC in accordance with 10 CFR 50.4, "Written Communications".

PBAPS

6.10 Record Retention

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principle maintenance activities, inspections, repair and replacement of principle items of equipment related to nuclear safety.
- c. Reportable Event Reports required by 10 CFR 50.73.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of reactor tests and experiments.
- f. Records of changes made to procedures required by Specification 6.8.
- g. Records of radioactive shipments.
- h. Records of sealed source leak tests and results.
- i. Records of annual physical inventory of all source material of record.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Record and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of facility radiation and contamination surveys.



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PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
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ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 171
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated July 13, 1990, as supplemented by letters dated November 27, 1991, and February 21, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter 1.
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 171, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Charles L. Miller

Charles L. Miller, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance. May 7, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 171

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix A Technical Specifications with the enclosed s. The revised areas are indicated by marginal lines.

<u>Page</u>	<u>Insert</u>
247	247
248	248
252a	252a
253	253
254	254
260	260

PBAPS

Meeting Frequency

- 6.5.1.4 The PORC shall meet at least once per calendar month and as convened by the PORC Chairman or his designated alternate(s).

Quorum

- 6.5.1.5 A quorum of the PORC necessary for the performance of the PORC responsibilities and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate(s) and four members or their alternates.

Responsibilities

- 6.5.1.6 The Plant Operations Review Committee shall be responsible for:
- a. Review of 1) Administrative procedures and changes thereto, 2) new procedures requiring a 10 CFR 50.59 safety evaluation, and 3) proposed changes to procedures requiring a 10 CFR 50.59 safety evaluation.
 - b. Review of all proposed tests and experiments that affect nuclear safety.
 - c. Review of all proposed changes to the Technical Specifications.
 - d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
 - e. Investigation of all violations of the Technical Specifications and shall prepare and forward a report covering evaluation and recommendations to prevent recurrence to the Plant Manager, the Vice President, Peach Bottom Atomic Power Station, and the Nuclear Review Board.
 - f. Review of facility operations to detect potential safety hazards.
 - g. Performance of special reviews and investigations and reports thereon as requested by the Chairman of the Nuclear Review Board.
 - h. Review of all reportable events required by 10 CFR 50.73.

PBAPS

6.5.1.6 Continued

- i. Review of the Plant Security Plan, and shall submit recommended changes to the Plan to the Plant Manager and the Nuclear Review Board.
- j. Review of the Emergency Plan, and shall submit recommended changes to the Plan to the Plant Manager and the Nuclear Review Board.
- k. Review of every unplanned release reportable under 10CFR 50.72 and 50.73, of radioactive material to the environs; evaluate the event; specify remedial action to prevent recurrence; and document the event description, evaluation, and corrective action and the disposition of the corrective action in the plant records.

Authority

6.5.1.7 The Plant Operations Review Committee shall:

- a. Recommend in writing, to the Plant Manager, for his approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(b) through (e) above constitutes an unreviewed safety question, as defined in 10 CFR 50.59.
- c. Provide immediate written notification to the Vice President, Peach Bottom Atomic Power Station, or in his absence, the Senior Vice President - Nuclear and the Nuclear Review Board of disagreement between the PORC and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Senior Vice President-Nuclear, and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.5.3 Procedure Review and Approval

- 6.5.3.1 Each new procedure and procedure change shall be reviewed by a Station Qualified Reviewer (SQR) and then by a Superintendent designated as responsible for the procedure. Administrative procedures (and changes thereto) shall be reviewed by PORC prior to approval by the Plant Manager or his designated alternate in accordance with Specification 6.1.1. The SQR shall not be the individual who prepared the new procedure or procedure change, he may however be from the same organization as the preparer. In addition to performing procedure reviews, the responsible Superintendents shall ensure that a sufficient complement of SQR's for their functional area is maintained. PORC shall have the option to review new procedures or procedure changes without the prior review and approval of the SQR and responsible Superintendent. Procedures reviewed in this manner will be approved by the Plant Manager.
- 6.5.3.2 Unless preempted by PORC as described in 6.5.3.1, the SQR shall review each new procedure or procedure change. The SQR shall render a determination in writing of whether or not a cross-disciplinary review of the new procedure or procedure change is necessary. If such a review is determined to be necessary, it shall be performed by the appropriate personnel. Upon completion of the SQR review, each new procedure or procedure change shall be reviewed by the Superintendent designated by Administrative procedures as the responsible Superintendent for that procedure. The Superintendent shall, as part of his review, decide whether or not a 10 CFR 50.59 Safety Evaluation is required. If a Safety Evaluation is not required, the new procedure or procedure change shall be approved by the responsible Superintendent or the Plant Manager prior to implementation.
- 6.5.3.3 If the responsible Superintendent determines that a new procedure or procedure change requires a 10 CFR 50.59 Safety Evaluation, the responsible Superintendent shall render a determination in writing of whether or not the new procedure or procedure change involves an Unreviewed Safety Question (USQ). The responsible Superintendent shall then forward the new procedure or procedure change with the associated Safety Evaluation to PORC for review. If an USQ is involved, prior NRC approval is required per 10 CFR 50.59 before implementation of the new procedure or procedure change.
- 6.5.3.4 Personnel recommended to perform the function of SQR shall be approved and designated as such by the PORC Chairman. The SQR shall be an individual from the responsible organization and knowledgeable in the functional area affected. The individual shall meet the qualifications of ANSI/ANS-3.1-1981, Selection, Qualification and Training of Personnel for Nuclear Power Plants, Section 4, excluding subsections 4.3.2 and 4.5. Individuals who do not possess the formal educational requirements specified shall not be automatically eliminated where other factors provide sufficient demonstration of their abilities. These factors, as listed in subsection 4.1 of ANSI/ANS-3.1, shall be evaluated on a case-by-case basis and approved and documented by the PORC Chairman.
- 6.5.3.5 Temporary procedure changes shall be reviewed and approved in accordance with Specification 6.8.3.
- 6.5.3.6 Records documenting the activities performed under Specification 6.5.3.1 through 6.5.3.5 shall be maintained in accordance with Specification 6.10.

PBAPS

6.6 Reportable Event Action

6.6.1 The following actions shall be taken for Reportable Events:

- a. The Commission shall be notified pursuant to the requirements of Section 50.73 to 10 CFR 50.
- b. Each Reportable Event Report submitted to the Commission shall be reviewed by the PORC and submitted to the NRB and the Vice President, Peach Bottom Atomic Power Station.

6.7 Safety Limit Violation

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Vice President, Peach Bottom Atomic Power Station, Plant Manager, and the NRB shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NRB and the Vice President, Peach Bottom Atomic Power Station within 10 working days of the violation.

6.8 Procedures

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained as follows, except as provided in Technical Specifications 6.8.2 and 6.8.3:

- a. Procedures that meet the requirements of Sections 5.1 and 5.3 of ANSI N18.7-1972
- b. Procedures that meet the requirements of Appendix "A" of USAEC Regulatory Guide 1.33 (November 1972)
- c. Procedures covering radiological effluent technical specification activities, including the Offsite Dose Calculation Manual and Quality Assurance Program for environmental monitoring using the guidance in Regulatory Guide 4.1, Revision 1, April 1975.

- 6.8.2 Each procedure and administrative policy of Technical Specification 6.8.1 and changes thereto, and any other procedure or procedure change that the Plant Manager determines to affect nuclear safety shall be reviewed and approved in accordance with Technical Specification 6.5.3 (and 6.5.1.6.a and 6.5.1.7.a, if required) prior to implementation. Each procedure of Technical Specification 6.8.1 shall also be reviewed periodically as set forth in Administrative procedures.
- 6.8.3 Temporary changes to procedures of Technical Specification 6.8.1 above may be made, provided:
- The intent of the original procedure is not altered.
 - The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
 - Within 14 days of implementation, the change is documented, reviewed by an SQR in accordance with Technical Specification 6.5.3.1, and approved by either the Plant Manager (or his designated alternate in accordance with Technical Specification 6.1.1) or the Superintendent designated by Administrative procedures as the responsible Superintendent for that procedure.

6.8.4 (Relocated into 6.8.1.c)

6.9 Reporting Requirements

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted to the NRC in accordance with 10 CFR 50.4, "Written Communications".

PBAPS

6.10 Record Retention

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principle maintenance activities, inspections, repair and replacement of principle items of equipment related to nuclear safety.
- c. Reportable Event Reports required by 10 CFR 50.73.
- d. Records of surveillance activities, inspections and operations required by these Technical Specifications.
- e. Records of reactor tests and experiments.
- f. Records of changes made to procedures required by Specification 6.8.
- g. Records of radioactive shipments.
- h. Records of sealed source leak tests and results.
- i. Records of annual physical inventory of all source material of record.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Record and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of facility radiation and contamination surveys.