

RLB-92-114

May 15, 1992

U. S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Reference: Quad Cities Nuclear Power Station Docket Number 50-265, DPR-30, Unit Two

Enclosed is Licensee Event Report (LER) 92-014, Re- on 00, for Quad Cities Nuclear Power Station.

This report is submitted in accordance with the requirements of the Code of Federal Regulations, Title 10, Part 50.73(a)(2)(ii). Any event or condition that resulted in the condition of the nuclear power plant, including its principal safety barriers, being seriously degraded, or that resulted in the nuclear plant being in a condition that was outside the design basis of the plant.

Respectfully,

COMMONWEALTH EDISON COMPANY QUAD CITIES NUCLEAR POWER STATION

R. L. Bax Station Manager

RIB/TB/plm

Enclosure

cc: J. Schrage T. Taylor INPO Records Center NRC Region III

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ABSTRACT:

On April 16, 1992, Unit Two was in the SHUTDOWN mode at zero percent of rated core thermal power. At that time, the Unit 1/2 Emergency Diesel Generator [DG] [EK] was declared inoperable to Unit Two due to the Unit 1/2 Diesel Generator Cooling Water Pump Room Cooler only having a single 480 volt feed from Switchgear 18-2 [SWGR]. The apparent cause of this event is due to inadequate design during installation of the flood-proof RHR Service Water Vaults in the early 1970's. Corrective actions involved the installation of partial modification MO4-2-92-0061. This partial modification involved installing a redundant power feed to the Unit 1/2 Diesel Generator Cooling Water Pump Cooler Fans [FAN] 1/2-5749A and 1/2-5749B. This report is being submitted in accordance with 10CFR50.73(a)(2)(ii).

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PLANT AND SYSTEM IDENTIFICATION:

General Electric - Boiling Water Reactor - 2511 MWt rated core thermal power.

EVENT ILENTIFICATION:

1/2 Diesel Generator INOP to Unit Two due to lack of redundant powerfeed from Unit Two to 1/2 Diesel Generator Cooling Water Pump Room Cooler Fans.

A. CONDITIONS PRIOR TO EVENT:

Unit: Two Reactor Mode: 1

Event Date: April 16, 1992 Event Time: 1515 Mode Name: SHUTDOWN

Power Level: 00%

This report was initiated by Deviation Report D-4-2-92-054.

SHUTDOWN Mode (1) - In this position, a reactor scram is initiated, power to the control rod drives is removed, and the reactor protection trip systems have been deenergized for 10 seconds prior to permissive for manual reset.

8. DESCRIPTION OF EVENT:

On April 16, 1992, at 1515 hours, Unit Two was in the SHUTDOWN mode at C percent of rated core thermal power. At that time, the Unit 1/2 Emergency Diesel Generator [DG] [EK] was declared inoperable to Unit Two due to the Unit 1/2 Diesel Generator Cooling Water Pump Room Cooler [CLR] only having a single 480 Volt feed from Switchgear 18-2 [SWGR]. This problem was recently identified while reviewing degraded voltage conserns. This problem was recently identified while receiving degraded voltage concerns. This problem was originally identified and assessed during the 1989 Diesel Generator Safety System Functional Inspection (SSFI) (NTS Item NO. 254-303-89-00103KA), prior to any official mandate of the Operability Assessment Procedure QE40.1. As a result of this problem, the station updated procedure QOA 6700-4, 480V BUS 18 FAILURE, to caution operators that the 1/2 Diesel Generator Cooling Water Pump Room Cooler Fans would not be available in the event of a power loss to Switchgear 18-2. The station also initiated a modification request to install a power feed from Unit Two Division I.

NRC notification via ENS was made in accordance with 10CFR50.72(b)(2)(1) at 1738 hour on April 16, 1992. This regulation requires the reporting of any event, found while the unit is shutdown, that it had been found while the unit was in operation, would have resulted in the nuclear plant, including its principal safety barriers being seriously degraded or being in an unanalyzed condition that significantly compromises plant safety.

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There were no other systems or components inoperable at the beginning of this event which could have contributed to the event.

C. APPARENT CAUSE OF EVENT:

This Licensee Event Report is being submitted in accordance with 10CFR50.73(a)(2)(ii), which requires reporting of any event or condition of the nuclear plant, including its principal safety barriers being seriously degraded or that resulted in the nuclear plant being in an unanalyzed condition that significantly compromises plant safety.

The apparent cause of this event is attributed to inadequate design during installation of the flood-proof RHR Service Water Vaults in the early 1970's. The 1/2 Diesel Generator Cooling Water Pump Room Coolers were installed during installation of the flood proof vaults. The Unit One and Unit Two Diesel Generator Cooling Water Pump Room Coolers were verified to have redundant feeds.

D. SAFETY ANALYSIS OF EVENT:

Quad Cities Unit Two Technical Specification 3.9.E.3 states, "when the reactor is in Cold Shutdown or Refueling mode, a minimum of one diesel generator (either the Unit diesel generator or the Unit 1/2 diesel generator) shall be operable whenever any work is being done which has the potential for draining the vessel, secondary containment is required, or a core or containment cooling system is required." Since the Unit Two Diesel Generator remained operable adequate power sources remained available to supply Division II loads if required. When discovered, the safety consequences of the event were minimal because Unit Two was in COLD SHUTDOWN.

Assuming a worse case single failure, the plan design is required to safely handle a Loss of Coolant Accident (LOCA) on one unit and provide normal shutdown of the other unit, coincident with a station Loss Of Offsite Power (LOOP). In the scenario with a Loss of Coolant Accident (LOCA) on Unit Two coincident with a loss of the Unit One 125 Volt DC battery system, the Unit Two Diesel Generator would fail to auto-start because of the loss of the Un't One 125 Volt DC battery system. The LCOA signal on Unit Two would cause the 1/2 Diesel Generator to load to Unit Two 4 KV Emergency Bus [BU] 23-1. Bus 28 would be available to power the Unit 1/2 Diesel Generator Cooling Water Pump, however, Switchgear 18-2 would not have a power source. Since the Unit 1/2 Diesel Generator Cooling Water Pump Room Cooler would not be powered, the Unit 1/2 Diesel Generator Cooling Water Pump would not receive adequate cooling and eventually fail. However, an alternate method of providing cooling to the 1/2 Diesel Generator Cooling Water Pump is to power Switchgear 18-2 by manually closing both Bus 18 and 19 cross-tie breakers. Procedure QOA 6700-4 directs the operators to take this action if Bus 18 cannot be energized from Bus 13-1. To ensure operability of the 1/2 Diesel Generator Cooling Water Pump, CECo Nuclear Engineering Department (NED) has determined that cooling must be provided within 30 minutes.

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TEXT Energy Industry Identi	fication System (EIIS) codes	are identified in the text as [XX]			

E. CORRECTIVE ACTIONS:

Corrective actions involved the design and installation of partial modification MO4-2-92-006I which upgraded the existing power feed cables for the Diesel Generator Cooling Water Pump Cooler Fans 1/2-5749A and 1/2-5749B. In addition, this partial modification provided a redundant power feed to the Unit 1/2 Diesel Generator Cooling Water Pump Room Cooler Fans A and B.

The change in power feed for the 1/2 Diesel Generator Cooling Water Pump Room Cooler Fans was accomplished by utilizing the same power feed supplies (both normal and alternate) that provide power for the 1/2 Diesel Generator Cooling Water Pump Motor. Isolation fuses were installed in the 2251-100 panel to assure that, if a cooling water pump room cooler fan malfunctions, the power feed to the cooling water pumps will not be degraded.

This partial modification was performed during the current Refuel Outage, Q2R11.

F. PREVIOUS EVENTS:

A search was performed on the systems included in the RHR Service Water Flood Proof Vaults. There have been no previous events involving inadequate design due to the installation of the flood proof vaults at Quad Cities Station.

G. COMPONENT FAILURE DATA:

There was no component failure identified with this event.