## REVISED COPY

UNIT SHUTDOWNS AND POWER REDUCTIONS REPORT FOR MARCH, 1992

No.	Date	Type	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code*	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
02-01	926301	S	744	С	4	NA	NA	NA	Unit offline continuing 1R10 refueling outage.

F: Forced

S: Scheduled

A-Equipment Failure (Explain)

B-Maintenance or Test

C-Refueling

D-kegulatory Restriction

E-Operator Training & License Examination

F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

3

Method:

1-Manual

2-Manual Scran.

3-Automatic Scram.

4-Continuation 5-Load Reduction

9-Other

24

Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee

Event Report (LER) File (NUREG-

50-313

ANO Unit 1

April 13, 1992

(501) 964-5535

K. R. Hayes

1022)

DOCKET NO.

UNIT NAME

DATE

TELEPHONE

COMPLETED BY

5...

Exhibit I - Same Source

9205210067 920515 PDR ADOCK 05000313

DATE: April, 1992

## REFUELING INFORMATION

1.	Name of facility: Arkansas Nuclear One - Unit 1						
2.	Scheduled date for next refueling shutdown. February 29, 1992						
3.	Scheduled date for restart following refueling, May 9, 1992						
4.	Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has 'be reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?						
	Yes. Technical Specification changes per GL 88-16 incorporating use of a Core Operating Limi's Report (COLR) was submitted to the NRC.						
5.	Scheduled date(s) for submitting proposed licensing action and supporting information. The COLR Technical Specification change request submitted to the NRC was approved April 17, 1992. The 50.59 for the Reload Report was approved by Plant Safety Committee for Cyc?a 11 operation						
6.	Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.						
	Non s.						
7.	The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 625*						
8.	The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planaed, in number of fuel assemblies.						
	present 968 increase size by 0						
9.	The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licased capacity.						
	DATE: 1995 (Loss of fullcore offload capability)						
	* Changed one to the addition of 60 discharged assemblies during 1R10.						