



January 17, 1996

Docket No. 50-423
B14873

Re: 10CFR50.90

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Millstone Nuclear Power Station, Unit No. 3
Proposed Revision to Operating License,
Deletion of License Condition Relating to
Salem ATWS Events, Generic Letter 83-28

Introduction

Pursuant to 10CFR50.90, Northeast Nuclear Energy Company (NNECO) hereby proposes to amend Operating License No. NPF-49 by deleting License Condition 2.C(4). NNECO has submitted responses to and implemented the requirements of Generic Letter (GL) 83-28.⁽¹⁾ These responses have been accepted by the NRC Staff; therefore, License Condition 2.C(4) is no longer necessary.

The proposed changes are discussed in detail below. Attachments 1 and 2 provide the marked-up and retyped pages of the Millstone Unit No. 3 Operating License, respectively.

Background

On July 8, 1983, the NRC issued GL 83-28 in response to the Salem anticipated transient without scram (ATWS) events. The GL identified the actions that the NRC Staff believed were necessary to respond to the ATWS events, and requested licensees furnish "the status of current conformance with the positions contained [within the GL], and plans and schedules for any needed improvements for conformance with the positions."

NNECO, on behalf of Millstone Unit No. 3, responded to this request in a number of submittals. The following paragraphs identify the actions required by GL 83-28, and the NRC Staff's transmittal which documents their acceptance of the Millstone Unit No. 3 responses.

(1) D. G. Eisenhut letter to All Licensees of Operating Reactors, Applicants for an Operating License, and Holders of Construction Permits, "Required Actions Based on Generic Implications of Salem ATWS Events (Generic Letter 83-28)," dated July 8, 1983.

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1.1 Post-Trip Review (Program Description and Procedure)

In Supplement No. 4 to NUREG-1431, "Safety Evaluation Report Related to the Operation of Millstone Nuclear Power Station, Unit No. 3,"⁽²⁾ the NRC Staff concluded that Millstone Unit No. 3's response to Action 1.1 was acceptable.

In a letter dated November 8, 1983,⁽³⁾ NNECO provided a commitment regarding the qualifications and training of Duty Officers. In a letter dated November 13, 1995,⁽⁴⁾ NNECO identified that this commitment had been revised in the pilot program to validate the Nuclear Energy Institute Guidelines for Managing NRC Commitments.

1.2 Post-Trip Review - Data and Information Capability

In a letter dated November 19, 1986,⁽⁵⁾ the NRC Staff concluded that Millstone Unit No. 3's response to Action 1.2 was acceptable.

2.1 Equipment Classification and Vendor Interface (Reactor Trip System Components)

In letters dated November 19, 1986, and August 4, 1987,⁽⁶⁾ the NRC Staff concluded that Millstone Unit No. 3's responses to parts 1 and 2 of Action 2.1 were acceptable.

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- (2) B. J. Youngblood letter to J. F. Opeka, "Issuance of Supplement No. 4 to NUREG-1431 - Millstone Nuclear Power Station, Unit No. 3," dated December 6, 1985.
 - (3) W. G. Council letter to D. G. Eisenhut, "Haddam Neck Plant, Millstone Nuclear Power Station, Unit Nos. 1, 2, and 3, Response to Generic Letter 83-28, Generic Implications of Salem ATWS Events," dated November 8, 1983.
 - (4) J. F. Opeka letter to the U.S. Nuclear Regulatory Commission, "Millstone Nuclear Power Station, Unit Nos. 1, and 3, Pilot Program Summary Report, NEI Guidelines for Managing NRC Commitments," dated November 13, 1995.
 - (5) E. L. Doolittle letter to J. F. Opeka, "Safety Evaluation Report for ATWS Items 1.2, 2.1 (Part 1), 3.1.3, 3.2.3, 4.1, 4.2.1, 4.2.2, and 4.3 - Technical Specification," dated November 19, 1986.
 - (6) R. L. Ferguson letter to E. J. Mroczka, "Generic Letter 83-28 ATWS Item 2.1 (Part 2) (TAC No. 60388)," dated August 4, 1987.

2.2 Equipment Classification and Vendor Interface (Programs for All Safety-Related Components)

In letters dated October 31, 1988,⁽⁷⁾ and May 3, 1991,⁽⁸⁾ the NRC Staff concluded that Millstone Unit No. 3's responses to parts 1 and 2 of Action 2.2 were acceptable.

3.1 Post-Maintenance Testing (Reactor Trip System Components)

In letters dated November 19, 1986, and June 26, 1987,⁽⁹⁾ the NRC concluded that Millstone Unit No. 3's responses to parts 1 through 3 of Action 3.1 were acceptable.

3.2 Post-Maintenance Testing (All Other Safety-Related Components)

In letters dated November 19, 1986, and June 26, 1987, the NRC Staff concluded that Millstone Unit No. 3's responses to parts 1 through 3 of Action 3.2 were acceptable.

4.1 Reactor Trip System Reliability (Vendor-Related Modifications)

In a letter dated November 19, 1986, the NRC concluded that Millstone Unit No. 3's response to Action 4.1 was acceptable.

4.2 Reactor Trip System Reliability (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)

In a letter dated November 19, 1986, the NRC concluded that Millstone Unit No. 3's response to parts 1 and 2 of Action 4.2 were acceptable. In Supplement 1 to GL 83-28,⁽¹⁰⁾ the NRC Staff informed

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- (7) J. F. Stolz letter to E. J. Mroczka, "Generic Letter 83-28, Item 2.2 Part 1 (TAC Nos. 53677, 53689, 53690, 60395)," dated October 31, 1988.
- (8) J. F. Stolz letter to E. J. Mroczka, "Haddam Neck and Millstone Unit Nos. 1, 2, and 3 - Vendor Interface for Safety-Related Components (TAC Nos. 76242/76257/76258/76259)," dated May 3, 1991.
- (9) R. L. Ferguson letter to E. J. Mroczka, "Generic Letter 83-28 ATWS Items 3.1.1, 3.1.2, 3.2.1, 3.2.2 and 4.5.1 (TAC Nos. 60389, 60396, 61400)," dated June 26, 1987.
- (10) J. G. Partlow letter to All Light-Water Reactor Licensees and Applicants, "Supplement 1 to Generic Letter 83-28, 'Required Actions Based on Generic Implications of Salem ATWS Events,'" dated October 7, 1992.

licensees that they were relaxing the original positions taken in parts 3 and 4 of Action 4.2. The NRC concluded that no licensee actions were necessary to respond to parts 3 and 4 of Action 4.2.

4.3 Reactor Trip System Reliability (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)

In a letter dated November 19, 1986, the NRC Staff stated that they had reviewed a copy of Table 3.3-1 of the Millstone Unit No. 3 Technical Specifications and had noticed that the table was not consistent with either the Westinghouse Standard Technical Specifications or GL 85-09. They stated that the action statements for functional unit no. 21 (reactor trip bypass breakers) were not appropriate, and recommended that functional unit no. 21 either be removed from the technical specifications or modified to be similar to functional unit no. 21 of Table 3.3-1 of the Shearon Harris Final Draft Technical Specifications.

On February 26, 1987,⁽¹¹⁾ NNECO proposed that functional unit no. 21 (reactor trip bypass breakers) be deleted from Tables 3.3-1 and 3.3-2. The NRC Staff approved this request in a letter dated August 7, 1987.⁽¹²⁾ No additional efforts are required for Millstone Unit No. 3 to conform with Action 4.3.

4.4 Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)

Action 4.4 is directed towards B&W plants. Millstone Unit No. 3 is a Westinghouse plant; therefore, Action 4.4 is not applicable to Millstone Unit No. 3.

(11) E. J. Mrocza letter to the U.S. Nuclear Regulatory Commission, "Millstone Nuclear Power Station, Unit No. 3, Proposed Revision to Technical Specifications, Reactor Trip Bypass Breakers," dated February 26, 1987.

(12) R. L. Ferguson letter to E. J. Mrocza, "Issuance of Amendment [Amendment No. 8] (TAC #64771)," dated August 7, 1987.

4.5 Reactor Trip System Reliability (System Functional Testing)

In letters dated February 24, 1987,⁽¹³⁾ June 26, 1987, and June 19, 1989,⁽¹⁴⁾ the NRC Staff concluded that Millstone Unit No. 3 responses to parts 1, 2, and 3 of Action 4.5 were acceptable.

Description of Proposed Changes

NNECO proposes to delete License Condition 2.C(4). NNECO's responses to the actions required by GL 83-28 have been accepted by the NRC Staff; therefore, License Condition 2.C(4) is no longer necessary.

Safety Assessment

NNECO's proposal to delete License Condition 2.C(4) is an administrative change. The proposed change does not affect the configuration, operation, or performance of any system, structure, or component.

Deletion of License Condition 2.C(4) is appropriate, because Millstone Unit No. 3 has submitted responses to and implemented the requirements of GL 83-28. The NRC Staff has accepted Millstone Unit No. 3's responses regarding the actions required by GL 83-28, thus, the license condition has been met and is no longer necessary.

Based on the above, deletion of License Condition 2.C(4) is appropriate and does not adversely affect public health and safety.

Significant Hazards Consideration

NNECO has reviewed the proposed change in accordance with 10CFR50.92 and concluded that the change does not involve a significant hazards consideration (SHC). The basis for this conclusion is that the three criteria of 10CFR50.92(c) are not compromised. The proposed change does not involve an SHC because the change would not:

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- (13) E. L. Doolittle letter to E. J. Mroczka, "Millstone Nuclear Power Station Unit No. 3, Generic Letter 83-28, Item 4.5.2, Reactor Trip System Reliability On-Line Testing," dated February 24, 1987.
- (14) D. H. Jaffe letter to E. J. Mroczka, "Safety Evaluation for Generic Letter 83-28, Item 4.5.3, Reactor Trip Reliability - On-Line Functional Testing of the Reactor Trip System (TAC No. 60401)," dated June 19, 1989.

1. Involve a significant increase in the probability or consequences of an accident previously analyzed.

NNECO's proposal to delete License Condition 2.C(4) is an administrative change. The NRC Staff has accepted Millstone Unit No. 3's responses regarding the actions required by GL 83-28, thus, the license condition has been met and is no longer necessary. The proposed change does not affect the configuration, operation, or performance of any system, structure, or component. Additionally, the limiting conditions for operation, limiting safety system settings, and safety limits specified in the Millstone Unit No. 3 Technical Specifications are unchanged. Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously analyzed.

2. Create the possibility of a new or different kind of accident from any previously analyzed.

The NRC Staff has accepted Millstone Unit No. 3's responses regarding the actions required by GL 83-28, thus, the license condition has been met and is no longer necessary. The proposed change to delete License Condition 2.C(4) does not affect the configuration, operation, or performance of any system, structure, or component. Additionally, the limiting conditions for operation, limiting safety system settings, and safety limits specified in the Millstone Unit No. 3 Technical Specifications are unchanged. Therefore, this proposed change cannot create the possibility of a new or different kind of accident from any previously analyzed.

3. Involve a significant reduction in the margin of safety.

The NRC Staff has accepted Millstone Unit No. 3's responses regarding the actions required by GL 83-28, thus, the license condition has been met and is no longer necessary. The proposed change to delete License Condition 2.C(4) does not affect the configuration, operation, or performance of any system, structure, or component. Additionally, the limiting conditions for operation, limiting safety system settings, and safety limits specified in the Millstone Unit No. 3 Technical Specifications are unchanged. Therefore, this proposed change does not involve a significant reduction in a margin of safety.

The Commission has provided guidance concerning the application of the standards of 10CFR50.92 by providing certain examples (51 FR 7751, March 6, 1986) of amendments that are not considered

likely to involve an SHC. NNECO's proposal to delete License Condition 2.C(4) is enveloped by example (i), "A purely administrative change to technical specifications: for example, a change to achieve consistency throughout the technical specifications, correction of an error, or a change in nomenclature."

Environmental Considerations

NNECO has reviewed the proposed license amendment against the criteria of 10CFR51.22 for environmental considerations. The proposed changes do not increase the types and amounts of effluents that may be released offsite, nor significantly increase individual or cumulative occupational radiation exposures. Based on the foregoing, NNECO concludes that the proposed changes meet the criteria delineated in 10CFR51.22(c)(9) for a categorical exclusion from the requirements for an environmental impact statement.

Nuclear Safety Assessment Board

The Nuclear Safety Assessment Board (formerly the Millstone Unit No. 3 Nuclear Review Board) has reviewed and concurred with the above determinations.

Commitments

There are no commitments contained within this submittal.

Schedule for NRC Approval and Issuance

No specific schedule for approval and issuance is requested. However, we request issuance of the license amendment at your earliest convenience with the amendment effective as of the date of issuance, to be implemented within 60 days of the date of issuance.

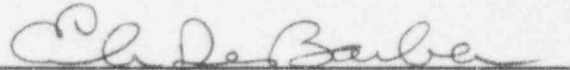
In accordance with 10CFR50.91(b), we are providing the State of Connecticut with a copy of this amendment.

U.S. Nuclear Regulatory Commission
B14873/Page 8
January 17, 1996

Should you have any questions regarding this submittal, please contact Mr. R. G. Joshi at (860) 440-2080.

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY



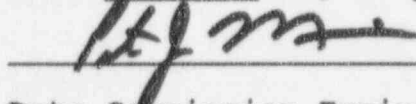
E. A. DeBarba
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cc: T. T. Martin, Region I Administrator
V. L. Rooney, NRC Project Manager, Millstone Unit No. 3
A. C. Cerne, Senior Resident Inspector, Millstone Unit No. 3

Mr. Kevin T.A. McCarthy, Director
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Subscribed and sworn to before me

this 17th day of January, 1996



PETER J. MINER
NOTARY PUBLIC

MY COMMISSION EXPIRES JUNE 30, 2000

Date Commission Expires: _____

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Attachment 1

Millstone Unit No. 3

Proposed Revision to Operating License,
Deletion of License Condition Relating to
Salem ATWS Events, Generic Letter 83-28
Marked-Up Pages

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