



Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038
 Hope Creek Generating Station

May 15, 1992

U. S. Nuclear Regulatory Commission
 Document Control Desk
 Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
 HOPE CREEK GENERATION STATION UNIT 1
 DOCKET NO. 50-354

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for April are being forwarded to you along with the summary of changes, tests, and experiments for April 1992 pursuant to the requirements of 10CFR50.59(b).

Sincerely yours,

J. J. Hagan,
 General Manager -
 Hope Creek Operations

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 Attachments
 C Distribution

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The Energy People

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-354
 UNIT Hope Creek
 DATE 5/15/92
 COMPLETED BY V. Zabielski
 TELEPHONE (609) 339-3506

MONTH April 1992

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>1060</u>	17.	<u>1052</u>
2.	<u>1061</u>	18.	<u>1064</u>
3.	<u>1061</u>	19.	<u>1053</u>
4.	<u>1061</u>	20.	<u>1053</u>
5.	<u>1009</u>	21.	<u>824</u>
6.	<u>1050</u>	22.	<u>486</u>
7.	<u>1046</u>	23.	<u>967</u>
8.	<u>1058</u>	24.	<u>987</u>
9.	<u>1049</u>	25.	<u>561</u>
10.	<u>1061*</u>	26.	<u>655</u>
11.	<u>1059*</u>	27.	<u>1058</u>
12.	<u>1037*</u>	28.	<u>1056</u>
13.	<u>1068*</u>	29.	<u>1059</u>
14.	<u>1072*</u>	30.	<u>1049</u>
15.	<u>1059*</u>	31.	<u>N/A</u>
16.	<u>1054</u>		

* A calculated value was used on April 10 through 15 due to the failure of the 'A' phase potential transformer, which was returned to service on the 15th.

OPERATING DATA REPORT

DOCKET NO. 50-354
 UNIT Hope Creek
 DATE 5/15/92
 COMPLETED BY V. Zabielski *VZ*
 TELEPHONE (609) 339-3506

OPERATING STATUS

1. Reporting Period April 1992 Gross Hours in Report Period 719
2. Currently Authorized Power Level (Mwt) 3293
 Max Depend. Capacity (MWe-Net) 1031
 Design Electrical Rating (MWe-Net) 1067
3. Power Level to which restricted (if any) (MWe-Net) None
4. Reasons for restriction (if any)
5. No. of hours reactor was critical

	<u>This Month</u>	<u>Yr To Date</u>	<u>Cumulative</u>
5. No. of hours reactor was critical	<u>719.0</u>	<u>2681.8</u>	<u>39,843.1</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
7. Hours generator on line	<u>719.0</u>	<u>2654.2</u>	<u>39,228.8</u>
8. Unit reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
9. Gross thermal energy generated (MWH)	<u>2,238,772</u>	<u>8,493,993</u>	<u>124,491,035</u>
10. Gross electrical energy generated (MWH)	<u>747,440</u>	<u>2,843,020</u>	<u>41,195,514</u>
11. Net electrical energy generated	<u>715,160</u>	<u>2,718,091</u>	<u>39,369,610</u>
12. Reactor service factor	<u>100.0</u>	<u>92.4</u>	<u>84.7</u>
13. Reactor availability factor	<u>100.0</u>	<u>92.4</u>	<u>84.7</u>
14. Unit service factor	<u>100.0</u>	<u>91.4</u>	<u>83.4</u>
15. Unit availability factor	<u>100.0</u>	<u>91.4</u>	<u>83.4</u>
16. Unit capacity factor (using MDC)	<u>96.5</u>	<u>90.8</u>	<u>81.2</u>
17. Unit capacity factor (Using Design MWe)	<u>93.2</u>	<u>87.8</u>	<u>78.5</u>
18. Unit forced outage rate	<u>0.0</u>	<u>0.0</u>	<u>4.9</u>
19. Shutdowns scheduled over next 6 months (type, date, & duration):
 Refueling outage, 9/12/92, 60 days
20. If shutdown at end of report period, estimated date of start-up:
 N/A

OPERATING DATA REPORT
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-354
UNIT Hope Creek
DATE 5/15/92
COMPLETED BY V. Zabielski
TELEPHONE (609) 339-3306

MONTH April 1992

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTION/COMMENTS
2	4/21	F	0	A	5	'B' Recirculation Pump Trip
3	4/25	F	0	A	5	Repairs made to 'B' Recirculation Motor/Generator Set

Summary

REFUELING INFORMATION

DOCKET NO. 50-354
 UNIT Hope Creek
 DATE 5/15/92
 COMPLETED BY S. Hollingsworth
 TELEPHONE (609) 339-1051

MONTH April 1992

1. Refueling information has changed from last month:

Yes No

2. Scheduled date for next refueling: 9/12/92

3. Scheduled date for restart following refueling: 11/11/92

4. A. Will Technical Specification changes or other license amendments be required?

Yes No

B. Has the reload fuel design been reviewed by the Station Operating Review Committee?

Yes No

If no, when is it scheduled? not scheduled (on or prior to 7/24/92)

5. Scheduled date(s) for submitting proposed licensing action: N/A

6. Important licensing considerations associated with refueling:

- Same fresh fuel as current cycle: no new considerations

7. Number of Fuel Assemblies:

A. Incore	<u>764</u>
B. In Spent Fuel Storage (prior to refueling)	<u>760</u>
C. In Spent Fuel Storage (after refueling)	<u>1008</u>

8. Present licensed spent fuel storage capacity: 4006

Future spent fuel storage capacity: 4006

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: 11/4, 2011
 (EOC16)
 (does not allow for full-core offload)

HOPE CREEK GENERATING STATION

MONTHLY OPERATING SUMMARY

April 1992

Hope Creek entered the month of April at approximately 100% power. On April 21, the 'B' Recirculation Pump tripped, causing a runback. The 'B' Recirculation Motor/Generator Set was repaired on April 25 and power was then returned to approximately 100% power. On April 30, the plant completed its 44th day of continuous power operation.

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE HOPE CREEK GENERATING STATION

APRIL 1992

The following items have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

DCP

Description of Safety Evaluation

4EC-3224

This DCP added three 30 amp single pole circuit breakers in a 120 volt AC power distribution panel and re-arranged branch circuits feeding space heaters in a motor control center. This allows the power feed for the Ultrasonic Resin Cleaner Panel to be re-routed to improve the voltage at the panel and enhance the efficiency of the Ultrasonic Resin Cleaner.

No Unreviewed Safety Questions were involved because this DCP involves only systems, equipment, and components that are non-safety related. Also, all of the materials that were used are compatible with previously used materials.

4EC-3251

This DCP installed two diodes, in series, parallel to the coils of the Intermediate Range Monitor/ Source Range Monitor alarm relays. The diodes were installed for noise suppression.

The diodes do not have any controlling functions nor do they affect the alarms or the Intermediate or Source Range Monitors. Therefore, no Unreviewed Safety Questions are involved with this DCP.

4EC-3341/02

This DCP provided permanent rigging points to allow the removal and reinstallation of Filtration, Recirculation, and Ventilation System backdraft isolation dampers in the Reactor Building.

This DCP does not involve work on equipment important to safety or items interfacing with equipment important to safety. Therefore, no Unreviewed Safety Question are involved with this DCP.

DCP

Description of Safety Evaluation

4EC-3342

During the Electrical Distribution Systems Functional Inspection, it was determined that the slow bus transfer scheme for the Class 1E 4KV buses could possibly transfer the loads from the normal infeed to the alternate infeed without stripping the loads. This could subject the motors on the 4KV buses to overvoltage transients. In addition, loads that had been sequenced onto a 4KV bus by the Loss Of Coolant Accident sequencer and were then stripped during a bus transfer might not be resequenced onto the bus after the alternate infeed closed. This DCP adds a time delay into the closing circuit of the alternate infeed breaker in the slow and dead bus transfer schemes to prevent the alternate infeed from closing too soon and to enable the sequencer to reset after a bus transfer.

The additional dead bus time ensures that the motors are not subjected to overvoltage transients when the alternate infeed closes. The additional dead bus time also ensures that the sequencer resets, allowing the loads to be re-sequenced onto the bus. The time delay also increases the reliability of the system. Therefore, no Unreviewed Safety Questions are involved with this DCP.

Procedure
Revision

Description of Safety Evaluation

NC.NA-AP.ZZ-0002(Q)
Rev 0

This new procedure reflects title changes and reporting relationships within the Nuclear Department.

The scope of this new procedure is limited to describing roles, responsibilities, and reporting relationships of key Nuclear Department personnel. This procedure does not change the design, function, or operation of any structures, systems, or components; therefore, there are no Unreviewed Safety Questions associated with this new procedure.

NC.NA-AP.ZZ-0013(Q)
Rev 1

This procedure revision identifies that temporary modifications made to operable components, systems, or structures to support the implementation of design changes, tests, and experiments are controlled by this procedure. It also states that space heaters and ventilation equipment is to be controlled by this procedure. Additionally, this revision changes some forms and part of the flowpath process.

This administrative procedure provides specific guidance for the implementation of the programs and processes described in the SAR. The proposed revision is in compliance with the SAR; therefore, there are no Unreviewed Safety Questions associated with this new procedure.

NC.NA-AP.ZZ-0055(Q)
Rev 1

This revision makes several changes to clarify the procedure as well as changes that are grammatical or editorial in nature.

This administrative procedure enhances the administrative control of outage management. The implementation of this procedure provides controls to ensure that outage activities are performed in compliance with licensing requirements and suggested industry guidelines. The proposed revision is in compliance with the SAR; therefore, there are no Unreviewed Safety Questions associated with this procedure revision.