

NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20655

May 5, 1992

Docket No. 50-271

Mr. L. A. Tremblay Senior Licensing Engineer Vermont Yankee Nuclear Power Corporation 580 Mair Street Bolton, Massachusetts 01740-1398

Dear Mr. Tremblay

SUBJECT: SAFETY EVALUATION FOR VERMONT YANKEE NUCLEAR POWER STATION RELAPSYA LOCA ANALYSIS METHODOLOGY (TAC NO. M74595)

The enclosed safety evaluation report prepared by the staff accepts for referencing the Topical Report YAEC-1547 as modified and supplemented during this review.

RELAP5YA, is based on RELAP5/MOD1 with modifications made by the Yankee Atomic Electric Company (YAEC) for use in Loss of Coolant Accident (LOCA) analyses for the Vermont Yankee Nuclear Power Station (VYNPS). RELAP5YA was first submitted on January 14, 1983 for staff review, as an evaluation model conforming to the criteria of Appendix K to 10 CFR 50. However, during the course of that review, the staff concluded that requirement II.5 of Appendix K was not satisfied. RELAP5YA was found acceptable for best estimate LOCA calculations over the entire break spectrum for the VYNPS.

The present review addresses the limitations discovered in the first review, which are related to compliance of the LOCA licensing analysis to conditions I.D.5 and I.D.6 of Appendix K. During this review the staff agreed that it was not necessary for YAEC to use heat transfer models conforming to Appendix K and that alternate heat transfer models would be acceptable provided that they was justified by comparison to appropriate experimental data. Thus, YAEC a mented the best-estimate, post-critical heat flux (CHF) heat transfer algor from in RELAP5/MOD1 with their own modifications and provided comparisons and qualification analysis to TLTA, FLECHT and THTF tests to support their acceptability. Based upon this review, the staff finds that the calculated wall temperatures are higher than the experimental data and correspondingly the computed heat transfer coefficients to be generally less than those measured during the tests. Therefore, we conclude the RELAP5YA provides sufficient assurance of conservatism as a licensing code for performing large BWR LOCA analyses.

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Mr. L. A. Tremblay

During this review there have been several supplemental submittals referring to corrections of previous information, or providing new information in response to staff requests. All these submittals are considered part of the original reports YAEC-1547 and YAEC-1300P. The staff recommends that the licensee incorporate all this information in a revised version of the report for ease and accessibility.

The staff's evaluation (Enclosure) has referenced the supplemental submittals noted above, and includes the contribution of ITS, our contractor for the RELAPSYA review. This completes the review for TAC No. M74595.

Sincerely,

original signed by:

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Patrick M. Sears, Project Manager Project Directorate I-3 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosure: Safety Evaluation for Vermont Yankee Nuclear Power Station RELAPSYA LOCA Analysis Methodology

cc w/enclosure: See next page

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Mr. L. A. Tremblay, Senior Licensing Engineer

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