



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

IOWA ELECTRIC LIGHT AND POWER COMPANY  
CENTRAL IOWA POWER COOPERATIVE  
CORN BELT POWER COOPERATIVE

DOCKET NO. 50-331

DUANE ARNOLD ENERGY CENTER

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 104  
License No. DPR-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Iowa Electric Light & Power Company, et al, dated January 27, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-49 is hereby amended to read as follows:

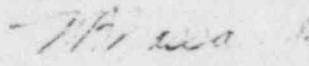
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 104, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 22, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 104

FACILITY OPERATING LICENSE NO. DPR-49

DOCKET NO. 50-331

Revise the Appendix "A" Technical Specifications by removing the current pages and inserting the revised pages listed below. The revised areas are identified by vertical lines.

LIST OF AFFECTED PAGES

3.2-23b

3.2-34a

## NOTES FOR TABLE 3.2-H

NOTES FOR TABLE 3.2-H

- (1) Each channel is comprised of three instruments (pressure switches) which are arranged in a "two out of three" logic connected to a relay.
- (2) From and after the date that a channel is inoperable, the torus temperature will be monitored at least once per shift to observe any unexplained temperature increase which might be indicative of an open SRV; continued reactor operation is permissible only during the succeeding 30 days, unless such channel is sooner made operable.
- (3) When the ability to obtain a sample has been lost:
  - a. Within 7 days confirm a sample can be obtained within 24 hours of the time a decision is made to sample; and
  - b. Within 90 days, restore the sampling capability.
  - c. If the requirements of notes 3 (a) and 3 (b) cannot be met, be in at least a HOT SHUTDOWN Condition within the next 24 hours
- (4) When the ability to analyze a sample has been lost:
  - a. Within 7 days, confirm that alternative sample analytical support services can be initiated within 24 hours of the time a decision is made to sample; and
  - b. Within 90 days, restore sample analysis capability.
  - c. If the requirements of notes 4(a) and 4(b) cannot be met, be in at least a HOT SHUTDOWN Condition within the next 24 hours.
- (5) With the number of operable channels (both indicator and recorder inoperable) less than the Minimum Channels Operable Requirement, initiate the preplanned alternate method of monitoring the appropriate parameter(s) within 72 hours, and:
  - a. either restore the inoperable channel(s) to operable status within seven (7) days following the event, or
  - b. prepare and submit a Special Report to the Commission within 14 days following the event describing the action taken, the cause of the inoperability and the plans and schedule for restoring the system to operable status.

TABLE 4.2-II

## ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>Instrument</u>	<u>Calibration Frequency</u>	<u>Instrument Check (2)</u>
Safety/Relief Valve Position Indicator (Primary) (1)(2)	Once/operating cycle	Once/month
Safety/Relief Valve Position Indicator (Backup-Thermocouple)	Once/operating cycle	Once/month
Safety Valve Position Indicator (Primary) (1)(2)	Once/operating cycle	Once/month
Safety Valve Position Indicator (Backup-Thermocouple)	Once/operating cycle	Once/month
Extended Range Effluent Radiation Monitors:		
a) Reactor Building Exhaust Stacks	Once/operating cycle (3)	Once/week
b) Turbine Building Exhaust Stack	Once/operating cycle (3)	Once/week
c) Offgas Stack	Once/operating cycle (3)	Once/week
Reactor Coolant, Containment Atmosphere, and Torus Water Post-Accident Sampling	Once/operating cycle (4)	N/A

NOTES FOR TABLE 4.2-H

1. Functional test of the relay is done once/3 months.
2. Instrument check shall consist of the qualitative assessment of channel behavior during operation by observation. This determination shall include, where possible, comparison of the channel indication and/or status with other indications and/or status derived from independent instrument channels (e.g. backup thermocouple) measuring the same parameter.
3. Accident range effluent monitors shall be calibrated by means of a built-in check source or a known radioactive source.
4. Not a calibration, but demonstration of system operability.