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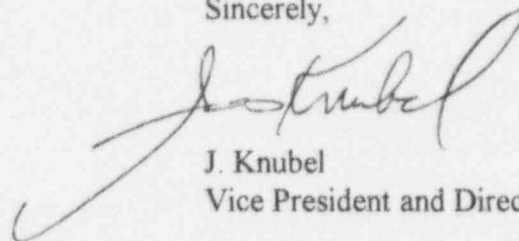
U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Monthly Operating Report for December 1995

Enclosed are two copies of the December 1995 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,



J. Knubel
Vice President and Director, TMI

9601190184 951231
PDR ADOCK 05000289
R PDR

WGH

Attachments

cc: Administrator, Region I
TMI Senior Resident Inspector
T95001

190049

IKRA
11

OPERATIONS SUMMARY December 1995

The plant entered the month operating at 100% power and remained at that power level for the remainder of the month. Net unit electrical output averaged approximately 820 MWe during December.

MAJOR SAFETY RELATED MAINTENANCE

The following is a summary of major safety related maintenance items accomplished during the month:

Makeup Pump MU-P-1A

Makeup Pump MU-P-1A was removed from service due to increased vibration. To alleviate the balance problem, split balance rings were installed on the exposed areas of the pump shaft between the bearing oil seals and the mechanical seals. Additional work performed included a visual inspection of the speed increaser gears, a coupling alignment check, and inboard and outboard bearing to shaft clearance checks. All inspection results were satisfactory. The pump was reassembled, tested per IST 1300-3H with satisfactory results and returned to service.

Reactor Coolant Recorder RC7TAR

Reactor Coolant Temperature Recorder RC7TAR was removed from service because the recorder was not advancing. During troubleshooting and corrective maintenance activities, the motor gear was found cracked and was replaced. RC7TAR operated correctly during post maintenance tests and was returned to service.

Diesel Driven Fire Pump FS-P-1

Diesel Driven Fire Service Pump FS-P-1 was removed from service due to the inoperability of the block heater. The block heater thermostat was found to be failed and was replaced. The maintenance effort included cleaning the heater elements and replacing a cracked flexible hose on the heater. Following the work, the block heater was tested and based on its satisfactory operation, FS-P-1 was returned to service.

Cation Demineralizer Piping

Weld repairs to two pin hole leaks in piping between the Cation Demineralizer and the Reactor Coolant Waste Evaporator were completed. The leaks were located at a welded coupling and a welded tee downstream of valves WDL-V-73 and WDL-V-74. They were determined to be thru wall leaks in the heat affected zone at the toe of the socket welds. The defects were excavated and liquid dye penetrant tested to ensure that they were completely removed. The areas were then welded and visually and PT inspected. After testing, the system was returned to service.

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE _____
 COMPLETED BY W G HEYSEK
 TELEPHONE (717) 948-8191

OPERATING STATUS

- 1. UNIT NAME: THREE MILE ISLAND UNIT 1
- 2. REPORTING PERIOD: DECEMBER 1995
- 3. LICENSED THERMAL POWER: 2568
- 4. NAMEPLATE RATING (GROSS MWe): 872
- 5. DESIGN ELECTRICAL RATING (NET MWe): 819
- 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 834
- 7. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 786

NOTES:

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS: _____

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): _____

10. REASONS FOR RESTRICTIONS, IF ANY: _____

		THIS MONTH	YR-TO-DATE	CUMMULATIVE
		-----	-----	-----
11. HOURS IN REPORTING PERIOD	(HRS)	744.0	8760.0	186985.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	(HRS)	744.0	7954.4	109543.1
13. REACTOR RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	2284.0
14. HOURS GENERATOR ON-LINE	(HRS)	744.0	7926.2	108381.3
15. UNIT RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED	(MWH)	1909359	20198655	266114926
17. GROSS ELECTRICAL ENERGY GENERATED	(MWH)	645634	6770220	89453474
18. NET ELECTRICAL ENERGY GENERATED	(MWH)	610193	6387986	84043404
19. UNIT SERVICE FACTOR	(%)	100.0	90.5	58.0
20. UNIT AVAILABILITY FACTOR	(%)	100.0	90.5	58.0
21. UNIT CAPACITY FACTOR (USING MDC NET)		104.3	92.8	57.2
22. UNIT CAPACITY FACTOR (USING DER NET)		100.1	89.0	54.9
23. UNIT FORCED OUTAGE RATE	(%)	0.0	0.0	35.9
UNIT FORCED OUTAGE HOURS	(HRS)	0.0	0.0	60761.2
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH):				

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
 UNIT TMI-1
 DATE _____
 COMPLETED BY W G HEYSEK
 TELEPHONE (717) 948-8191

MONTH: DECEMBER

DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)
1	820	17	824
2	821	18	825
3	822	19	816
4	820	20	818
5	824	21	816
6	824	22	815
7	824	23	816
8	826	24	817
9	825	25	817
10	827	26	816
11	824	27	814
12	819	28	815
13	817	29	818
14	819	30	820
15	823	31	820
16	823		

REPORT MONTH December 1995

DOCKET NO. 50-289
 UNIT NAME TMI-1
 DATE
 COMPLETED BY W. G. Heysek
 TELEPHONE (717) 948-8191

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report#	System Code ⁴ & ⁴	Component Code ¹ & ⁴	Cause & Corrective Action to Prevent Recurrence
						None			

1
F Forced
S Scheduled

2
 Reason
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
 Method
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
 Exhibit G - Instructions for preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5 Exhibit I same source

6 Actually used exhibits F & II NUREG 0161

REFUELING INFORMATION REQUEST

1. Name of Facility: **Three Mile Island Nuclear Station, Unit 1**
2. Scheduled date for next refueling shutdown: **September 12, 1997**
3. Scheduled date for restart following current refueling: **NA**
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? **NA.**
5. Scheduled date(s) for submitting proposed licensing action and supporting information: **NA**
6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures: **None.**
7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) **177** (b) **683**
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 1990. Phase 1 of the reracking project to increase spent fuel pool storage capacity permits storage of 1342 assemblies. Upon completion of Phase II of the reracking project, the full licensed capacity will be attained. Phase II is expected to be started in 2002.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Completion of Phase I of the reracking project permits full core off-load (177 fuel assemblies) through the end of Cycle 14 and on completion of the rerack project full core off-load is assured through the end of the current operating license and beyond.