



Commonwealth Edison  
LaSalle County Nuclear Station  
2601 N. 21st. Rd.  
Marseilles, Illinois 61341  
Telephone 815/357-6761

May 11, 1992

Director of Nuclear Reactor Regulation  
United States Nuclear Regulatory Commission  
Mail Station P1-137  
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for April, 1992.

Very truly yours,

*for* G. J. Diederich  
Station Manager  
LaSalle County Station

GJD/MJC/djf

Enclosure

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LASALLE NUCLEAR POWER STATTON  
UNIT 1  
MONTHLY PERFORMANCE REPORT  
APRIL 1992  
COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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(UNIT 1)

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## I. INTRODUCTION (Unit 1)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

## II. MONTHLY REPORT

### A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

| <u>Day</u> | <u>Time</u> | <u>Event</u>  |
|------------|-------------|---|
| 1          | 0000        | Reactor critical, Generator on-line at 1130 Mwe.  |
| 3          | 0200        | Reduced power level to 1000 Mwe due to system load.   |
|            | 1000        | Increased power level to 1130 Mwe.  |
| 4          | 0200        | Reduced power level to 1025 Mwe to transfer the Heater Drain system pumps.                          |
|            | 0600        | Increased power level to 1130 Mwe.  |
|            | 2300        | Reduced power level to 1025 Mwe to transfer the Heater Drain system pumps.                          |
| 5          | 0500        | Increased power level to 1130 Mwe.  |
| 7          | 0200        | Reduced power level to 950 Mwe to perform monthly surveillances.                                    |
|            | 1000        | Increased power level to 1135 Mwe.  |
| 11         | 0330        | Reduced power level to 900 Mwe to perform rod set.  |
|            | 1130        | Increased power level to 1130 Mwe.  |
| 14         | 0200        | Reduced power level to 1030 Mwe to transfer the Condensate/Condensate Booster pumps.                |
|            | 0930        | Increased power level to 1130 Mwe.  |
| 21         | 0130        | Reduced power level to 1020 Mwe to transfer the Condensate/Condensate Booster pumps.                |
|            | 1000        | Increased power level to 1130 Mwe.  |
| 22         | 1800        | Reduced power level to 990 Mwe to place the Motor Driven Reactor Feed Pump on line for Maintenance. |
|            | 2330        | Increased power level to 1130 Mwe.  |
| 25         | 0330        | Reduced power level to 995 Mwe due to system load.  |
|            | 1000        | Increased power level to 1130 Mwe.  |
| 27         | 0200        | Reduced power level to 995 Mwe due to system load.  |
|            | 1000        | Increased power level to 1130 Mwe.  |

## II. MONTHLY REPORT (CONTINUED)

### A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

| <u>Day</u> | <u>Time</u> | <u>Event</u>  |
|------------|-------------|---|
| 28         | 0330        | Reduced power level to 1000 Mwe due to system load. |
|            | 1000        | Increased power level to 1130 Mwe.                  |
|            | 2300        | Reduced power level to 1000 Mwe due to system load. |
| 29         | 0220        | Reduced power level to 870 Mwe due to system load.  |
|            | 1200        | Increased power level to 1130 Mwe.                  |
|            | 2300        | Reduced power level to 850 Mwe due to system load.  |
| 30         | 2400        | Reactor critical, Generator on-line at 850 Mwe.     |

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

(None.)

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including SOR differential pressure switch failure reports).  
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 1)

| <u>LER Number</u> | <u>Date</u> | <u>Description</u>  |
|-------------------|-------------|---|
| 92-005-00         | 04/06/92    | During performance of the Reactor Core Isolation Cooling system cold quick start surveillance, the RCIC turbine tripped on overspeed during the initial start sequence.   |
| 92-006-00         | 04/27/92    | Per engineering evaluations, the design setpoint of the 4KV degraded voltage relays may not be sufficient to assure operation and protection of safety related equipment. |

E. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 2)
2. Average Daily Unit Power Level (See Table 3)
3. Unit Shutdowns and Significant Power Reductions (See Table 4)

## C. TABLE 1 (Unit 1)

MAJOR CORRECTIVE MAINTENANCE TO  
SAFETY-RELATED EQUIPMENT

| WORK REQUEST<br>NUMBER | COMPONENT   | CAUSE OF MALFUNCTION                  | RESULTS AND EFFECTS<br>ON SAFE PLANT OPERATION               | CORRECTIVE ACTION                |
|------------------------|---|---------------------------------------|--|----------------------------------|
| L10727                 | Auxiliary Electric<br>Equipment Room<br>Ventilation Compressor<br>Motor CVE04CA | Oil temperature indicating<br>switch. | Inaccurate temperature readings.                             | Replaced temperature<br>switch.  |
| L14938                 | Reactor Core Isolation<br>Cooling System Turbine<br>1E51-C002                   | Governor valve 1E51-361.              | Turbine tripped on overspeed<br>during surveillance testing. | Replaced valve plug and<br>stem. |
| L15272                 | Control Room HVAC<br>Ammonia Detection<br>System                                | Ammonia detector<br>OXY-VC125B.       | Detector alarming below alarm<br>setpoint.                   | Rebuilt detector.                |
| L15311                 | Standby Gas Treatment<br>Equipment Train<br>Cooling Fan 1VG02C                  | 74 relay.                             | Interference relay operation.                                | Replaced relay.                  |

(No SOR Failures this month.)



TABLE 2  
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373  
UNIT LASALLE ONE  
DATE May 11, 1992  
COMPLETED BY M.J.CIALKOWSKI  
TELEPHONE (815) 357-6761

OPERATING STATUS

- |  |            |
|--|------------|
| 1. REPORTING PERIOD:                         | April 1992 |
| GROSS HOURS IN REPORTING PERIOD:             | 719        |
| 2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt):   | 3,323      |
| MAX DEPENDABLE CAPACITY (Mwe-Net):           | 1,036      |
| DESIGN ELECTRICAL RATING (Mwe-Net):          | 1,078      |
| 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY): | N/A        |
| 4. REASON FOR RESTRICTION (IF ANY):          |            |

|   | THIS MONTH                        | YEAR-TO-DATE | CUMULATIVE  |
|---|-----------------------------------|--------------|-------------|
|   | -----                             | -----        | -----       |
| 5. REACTOR CRITICAL TIME (HOURS)  | 719.0                             | 2,841.5      | 50,198.0    |
| 6. REACTOR RESERVE SHUTDOWN TIME (HOURS)  | 0.0                               | 0.0          | 1,641.2     |
| 7. GENERATOR ON-LINE TIME (HOURS)   | 719.0                             | 2,808.8      | 49,187.1    |
| 8. UNIT RESERVE SHUTDOWN TIME (HOURS)   | 0.0                               | 0.0          | 1.0         |
| 9. THERMAL ENERGY GENERATED (MWh <sub>t</sub> )                                   | 2,359,085                         | 9,098,837    | 144,255,816 |
| 10. ELECTRICAL ENERGY GENERATED (MWhe-Gross)                                      | 806,389                           | 3,108,222    | 48,198,004  |
| 11. ELECTRICAL ENERGY GENERATED (MWhe-Net)  | 779,720                           | 3,014,071    | 46,209,216  |
| 12. REACTOR SERVICE FACTOR (%)  | 100.0                             | 97.9         | 68.4        |
| 13. REACTOR AVAILABILITY FACTOR (%)   | 100.0                             | 97.9         | 70.7        |
| 14. UNIT SERVICE FACTOR (%)   | 100.0                             | 96.8         | 67.0        |
| 15. UNIT AVAILABILITY FACTOR (%)  | 100.0                             | 96.8         | 67.0        |
| 16. UNIT CAPACITY FACTOR (USING MDC) (%)  | 104.7                             | 92.3         | 60.6        |
| 17. UNIT CAPACITY FACTOR (USING DESIGN MWe)                                       | 101.6                             | 96.6         | 58.3        |
| 18. UNIT FORCED OUTAGE FACTOR (%)   | 0.0                               | 3.2          | 7.4         |
| 19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH): | Refuel Outage, 09/26/92, 12 Weeks |              |             |
| 20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:               |                                   |              |             |

TABLE 3  
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373  
UNIT LASALLE ONE  
DATE May 11, 1992  
COMPLETED BY M.J.CIALKOWSKI  
TELEPHONE (815) 357-6761

REPORTING PERIOD: April 1992

| DAY | POWER | DAY | POWER |
|-----|-------|-----|-------|
| --- | ----- | --- | ----- |
| 1   | 1,096 | 17  | 1,100 |
| 2   | 1,098 | 18  | 1,098 |
| 3   | 1,078 | 19  | 1,094 |
| 4   | 1,082 | 20  | 1,094 |
| 5   | 1,048 | 21  | 1,085 |
| 6   | 1,098 | 22  | 1,082 |
| 7   | 1,069 | 23  | 1,095 |
| 8   | 1,093 | 24  | 1,094 |
| 9   | 1,095 | 25  | 1,077 |
| 10  | 1,092 | 26  | 1,094 |
| 11  | 1,050 | 27  | 1,075 |
| 12  | 1,095 | 28  | 1,082 |
| 13  | 1,097 | 29  | 1,034 |
| 14  | 1,085 | 30  | 1,012 |
| 15  | 1,099 |     |       |
| 16  | 1,098 |     |       |

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%  
(Unit 1)

| YEARLY<br>SEQUENTIAL<br>NUMBER | DATE<br>(YYMMDD) | TYPE<br>F: FORCED<br>S: SCHEDULED | DURATION<br>(HOURS) | REASON | METHOD OF<br>SHUTTING DOWN<br>THE REACTOR OR<br>REDUCING POWER | CORRECTIVE<br>ACTIONS/COMMENTS<br>(LER/DVR # if<br>applicable) |
|--------------------------------|------------------|-----------------------------------|---------------------|--------|--|--|
| (None.)                        |                  |                                   |                     |        |  |  |

## SUMMARY OF OPERATION:

The unit remained on line at high power throughout the month. Several minor power reductions were required due to system loading and maintenance activities.

F. UNIQUE REPORTING REQUIREMENTS (Unit 1)

1. Safety/Relief valve operations  
(None.)
2. ECCS System Outages  
(See Table 5)
3. Changes to the Off-Site Dose Calculation Manual  
(None.)
4. Major changes to Radioactive Waste Treatment Systems.  
(None.)
5. Indications of Failed Fuel Elements.  
(None.)

(Unit 1)  
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

| <u>OUTAGE NO.</u> | <u>EQUIPMENT</u> | <u>PURPOSE</u>                     |
|-------------------|------------------|------------------------------------|
| (U-0)             |                  |                                    |
| 0269<br>0284      | ODG01K           | Check valve inspection.            |
| 0288              | ODG01K           | Disassemble and clean drains.      |
| (U-1)             |                  |                                    |
| 0179              | 1E22-S001        | Fuel oil suction valve inspection. |
| 0226              | 1E21-C003        | Coupling lubrication.              |
| 0237              | 1E51-C002        | Governor valve inspection.         |
| 0242              | 1E51-C002        | Change oil and filters.            |
| 0250              | 1E22-C302A       | Inspection.                        |
| 0252              | 1E22-F001        | Wiring inspection.                 |
| 0269              | 1DG03CB          | Valve inspection.                  |
| 0270              | 1E12-F052B       | EQ inspection.                     |
| 0280              | 1DG08CA          | Valve inspection.                  |
| 0286              | 1DG01K           | Inspection.                        |

LASALLE NUCLEAR POWER STATION  
UNIT 2  
MONTHLY PERFORMANCE REPORT  
APRIL 1992  
COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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(Unit 2)

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- D. LICENSEE EVENT REPORTS
- E. DATA TABULATIONS
  - 1. Operating Data Report
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  - 3. Unit Shutdowns and Power Reductions
- F. UNIQUE REPORTING REQUIREMENTS
  - 1. Safety/Relief Valve Operations
  - 2. ECCS System Outages
  - 3. Off-Site Dose Calculation Manual Changes
  - 4. Major Changes to Radioactive Waste Treatment System
  - 5. Indications of Failed Fuel Elements

## I. INTRODUCTION (Unit 2)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and flowdown. The architect-engineer was Sargent and Lundy and the primary construction contract was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761 extension 2427.



## II. MONTHLY REPORT

### A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

| <u>Day</u> | <u>Time</u> | <u>Event</u>  |
|------------|-------------|---|
| 1          | 0000        | Reactor subcritical, Generator off-line, Refuel outage (L2R04) in progress.                                     |
| 2          | 1814        | Reactor critical.   |
|            | 2144        | Reactor subcritical, all control rods inserted due to problems with the Reactor Water Cleanup isolation valves. |
| 7          | 2330        | Reactor critical.   |
| 12         | 1907        | Generator on-line.  |
|            | 2043        | Manual Turbine trip due to high vibrations caused by rubs on the recently installed turbine rotors.             |
| 13         | 0420        | Generator on-line.  |
|            | 0640        | Manual Turbine trip due to high vibrations caused by rubs on the recently installed turbine rotors.             |
|            | 1335        | Generator on-line.  |
| 14         | 0200        | Increased power level to 225 Mwe.   |
| 15         | 1509        | Manual Reactor scram due to erratic behavior of the Main Turbine Electro-Hydraulic Control system.              |
| 19         | 0555        | Reactor critical.   |
|            | 2230        | Generator on-line.  |
| 20         | 0231        | Manual Turbine trip to perform overspeed testing.   |
|            | 0404        | Generator on-line.  |
| 21         | 1345        | Increased power level to 225 Mwe.   |
|            | 1730        | Increased power level to 315 Mwe.   |
|            | 2045        | Increased power level to 400 Mwe.   |
|            | 2205        | Increased power level to 524 Mwe.   |
| 22         | 1030        | Increased power level to 833 Mwe.   |
| 24         | 2130        | Increased power level to 900 Mwe.   |

## II. MONTHLY REPORT (CONTINUED)

### A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

| <u>Day</u> | <u>Time</u> | <u>Event</u>   |
|------------|-------------|--|
| 25         | 0300        | Increased power level to 1000 Mwe (level held due to the 'A' Turbine Driven Reactor Feed Pump Being Out Of Service for maintenance). |
| 27         | 1000        | Reduced power level to 950 Mwe due to low Condensate Booster pump suction pressure.  |
|            | 1600        | Increased power level to 1000 Mwe.   |
| 30         | 2400        | Reactor critical, Generator on-line at 1000 Mwe.   |

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

(None.)

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY RELATED EQUIPMENT (including SOR differential pressure switch failure reports).  
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 2)

| <u>LER Number</u> | <u>Date</u> | <u>Description</u>  |
|-------------------|-------------|---|
| 92-004-00         | 04/15/92    | Manual Reactor scram caused by the Turbine Bypass Valves cycling excessively due to a Electro-Hydraulic Control system malfunction. |
| 92-006-00         | 04/20/92    | Reactor Water Cleanup system isolation caused by high differential flow.  |

E. DATA TABULATIONS (Unit 2)

1. Operating Data Report.  
(See Table 2)
2. Average Daily Unit Power Level.  
(See Table 3)
3. Unit Shutdowns and Significant Power Reductions.  
(See Table 4)

C. TABLE 1 (Unit 2)

MAJOR CORRECTIVE MAINTENANCE TO  
SAFETY-RELATED EQUIPMENT

| <u>WORK REQUEST<br/>NUMBER</u> | <u>COMPONENT</u>                | <u>CAUSE OF MALFUNCTION</u> | <u>RESULTS AND EFFECTS<br/>ON SAFE PLANT OPERATION</u> | <u>CORRECTIVE ACTION</u> |
|--------------------------------|---------------------------------|-----------------------------|--|--------------------------|
| L10025                         | Control Rod Drive<br>42-47      | Valve positioning.          | Control rod double notched.                            | Replaced valve.          |
| L11386                         | Hydraulic Control<br>Unit 14-15 | Water accumulator.          | Frequent water alarms.                                 | Replaced accumulator.    |
| L12099                         | Hydraulic Control<br>Unit 34-11 | Water accumulator.          | Frequent water alarms.                                 | Replaced accumulator.    |
| L12148                         | Hydraulic Control<br>Unit 22-19 | Water accumulator.          | Frequent water alarms.                                 | Replaced accumulator.    |
| L12149                         | Control Rod Drive<br>06-31      | Unit degradation.           | Degraded operation.                                    | Replaced control rod.    |
| L12150                         | Control Rod Drive<br>10-39      | Unit degradation.           | Degraded operation.                                    | Replaced control rod.    |
| L12151                         | Control Rod Drive<br>42-31      | Unit degradation.           | Degraded operation.                                    | Replaced control rod.    |
| L12152                         | Control Rod Drive<br>38-43      | Unit degradation.           | Degraded operation.                                    | Replaced control rod.    |
| L12153                         | Control Rod Drive<br>34-59      | Unit degradation.           | Degraded operation.                                    | Replaced control rod.    |
| L12154                         | Control Rod Drive<br>34-59      | Unit degradation.           | Degraded operation.                                    | Replaced control rod.    |
| L12155                         | Control Rod Drive<br>22-19      | Unit degradation.           | Degraded operation.                                    | Replaced control rod.    |
| L12156                         | Control Rod Drive<br>14-27      | Unit degradation.           | Degraded operation.                                    | Replaced control rod.    |

## C. TABLE 1 (Unit 2) —CONTINUED

MAJOR CORRECTIVE MAINTENANCE TO  
SAFETY-RELATED EQUIPMENT

| <u>WORK REQUEST<br/>NUMBER</u> | <u>COMPONENT</u>                          | <u>CAUSE OF MALFUNCTION</u>                 | <u>RESULTS AND EFFECTS<br/>ON SAFE PLANT OPERATION</u> | <u>CORRECTIVE ACTION</u>                          |
|--------------------------------|---|---|--|---|
| L13673                         | Safety Relief Valve<br>2B21-F013C         | 'A' solenoid electrical<br>connection.      | None.  | Replaced solenoid.                                |
| L13820                         | Hydraulic Control<br>Unit 10-27           | Valve leakage.                              | Degraded operation.                                    | Replaced valve.                                   |
| L13926                         | Hydraulic Control<br>Unit 42-11           | Water accumulator.                          | Frequent water alarms.                                 | Replaced accumulator.                             |
| L14467                         | Hydraulic Control<br>Unit 42-55           | Valve leakage.                              | Degraded operation.                                    | Replaced valve.                                   |
| L14468                         | Hydraulic Control<br>Unit 22-27           | Valve leakage.                              | Degraded operation.                                    | Replaced valve.                                   |
| L14469                         | Hydraulic Control<br>Unit 18-07           | Valve leakage.                              | Degraded operation.                                    | Replaced valve.                                   |
| L14470                         | Hydraulic Control<br>Unit 14-07           | Valve leakage.                              | Degraded operation.                                    | Replaced valve.                                   |
| L14723                         | Hydraulic Control<br>Unit 10-47           | Valve leakage.                              | Degraded operation.                                    | Replaced valve.                                   |
| L14838                         | Division II Battery<br>Charger 2DC17E     | Current sensing and<br>amplifier boards.    | Charger putting out low voltage.                       | Replaced current sensing<br>and amplifier boards. |
| L15162                         | Hydraulic Control<br>Unit 06-27           | Valve leakage.                              | Degraded operation.                                    | Replaced valve.                                   |
| L15184                         | Hydraulic Control<br>Unit 38-11           | Valve leakage.                              | Degraded operation.                                    | Replaced valve.                                   |
| L15421                         | Residual Heat Removal<br>Shutdown Cooling | Differential pressure<br>switch 2E31-N012AA | Switch exceeded rejection limits<br>during testing.    | Replaced switch.                                  |

## C. TABLE 1 (Unit 2) —CONTINUED

MAJOR CORRECTIVE MAINTENANCE TO  
SAFETY-RELATED EQUIPMENT

| <u>WORK REQUEST<br/>NUMBER</u> | <u>COMPONENT</u>                | <u>CAUSE OF MALFUNCTION</u> | <u>RESULTS AND EFFECTS<br/>ON SAFE PLANT OPERATION</u> | <u>CORRECTIVE ACTION</u> |
|--------------------------------|---------------------------------|-----------------------------|--|--------------------------|
| L97545                         | Hydraulic Control<br>Unit 02-23 | Water accumulator.          | Frequent water alarms.                                 | Replaced accumulator.    |
| L97549                         | Hydraulic Control<br>Unit 02-35 | Water accumulator.          | Frequent water alarms.                                 | Replaced accumulator.    |
| L97553                         | Hydraulic Control<br>Unit 30-27 | Water accumulator.          | Frequent water alarms.                                 | Replaced accumulator.    |
| L97554                         | Hydraulic Control<br>Unit 34-07 | Water accumulator           | Frequent water alarms.                                 | Replaced accumulator.    |
| L97572                         | Hydraulic Control<br>Unit 26-35 | Water accumulator.          | Frequent water alarms.                                 | Replaced accumulator.    |
| L99501                         | Hydraulic Control<br>Unit 54-31 | Valve leakage.              | Degraded operation.                                    | Replaced valve.          |
| L99502                         | Hydraulic Control<br>Unit 38-51 | Valve leakage.              | Degraded operation.                                    | Replaced valve.          |
| L99553                         | Control Rod Drive<br>26-27      | Seal leakage.               | Degraded operation.                                    | Replaced seals.          |
| L99554                         | Control Rod Drive<br>46-35      | Seal leakage.               | Degraded operation.                                    | Replaced seals.          |

(See attached SOR Failure Report.)

SOR dp SWITCH FAILURE DATA SHEET

Equipment Piece Number: 2E31-N012AA

Model Number: 103-AS-B203-NX-JJTTX6

Serial Number: 90-8-6434

Application: RHR Shutdown Cooling High Suction Flow Isolation

Date and Time of Discovery: 04/29/92 1345 hours

Reactor Mode: 1 (Run) Power Level: 87%

Calibration Tolerance: 167.4 - 169.4 "WC

Nominal Setpoint: <168.4 "WC

Action Limits: <163.0 or >173.8 "WC

Reject Limits: <158.8 or >178.0 "WC

Technical Specification

Limits: 186.0 "WC

As Found Setpoint: 156.5 "WC

Date and Time of Return to Service: 04/30/92 0845 hours

Model Number of Replacement Switch: 103-AS-B203-NX-JJTTX6

Serial Number of Replacement Switch: 90-8-6433

DVR Number: 1-2-92-051

Cause: Switch was found out of calibration. Apparent cause is instrument drift.

Corrective Action: The switch was replaced.

TABLE 2  
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373  
UNIT LASALLE TWO  
DATE May 11, 1992  
COMPLETED BY M.J.CIALKOWSKI  
TELEPHONE (815) 357-6761

OPERATING STATUS

- |  |            |
|--|------------|
| 1. REPORTING PERIOD:                         | April 1992 |
| GROSS HOURS IN REPORTING PERIOD:             | 719        |
| 2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt):   | 3,323      |
| MAX DEPENDABLE CAPACITY (Mwe-Net):           | 1,036      |
| DESIGN ELECTRICAL RATING (Mwe-Net):          | 1,078      |
| 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY): | N/A        |
| 4. REASON FOR RESTRICTION (IF ANY):          |            |

|   | THIS MONTH | YEAR-TO-DATE | CUMULATIVE  |
|---|------------|--------------|-------------|
|   | -----      | -----        | -----       |
| 5. REACTOR CRITICAL TIME (HOURS)  | 469.2      | 553.2        | 45,489.0    |
| 6. REACTOR RESERVE SHUTDOWN TIME (HOURS)  | 0.0        | 0.0          | 1,716.9     |
| 7. GENERATOR ON-LINE TIME (HOURS)   | 317.5      | 391.4        | 44,596.0    |
| 8. UNIT RESERVE SHUTDOWN TIME (HOURS)   | 0.0        | 0.0          | 0.0         |
| 9. THERMAL ENERGY GENERATED (MWh <sub>t</sub> )                                   | 667,956    | 848,942      | 133,091,341 |
| 10. ELECTRICAL ENERGY GENERATED (MWh <sub>e</sub> -Gross)                         | 222,732    | 287,006      | 44,176,756  |
| 11. ELECTRICAL ENERGY GENERATED (MWh <sub>e</sub> -Net)                           | 209,025    | 246,745      | 42,378,249  |
| 12. REACTOR SERVICE FACTOR (%)  | 65.3       | 19.2         | 68.7        |
| 13. REACTOR AVAILABILITY FACTOR (%)   | 65.3       | 19.2         | 71.5        |
| 14. UNIT SERVICE FACTOR (%)   | 44.2       | 13.6         | 67.8        |
| 15. UNIT AVAILABILITY FACTOR (%)  | 44.2       | 13.6         | 67.8        |
| 16. UNIT CAPACITY FACTOR (USING MDC) (%)  | 28.1       | 8.3          | 62.3        |
| 17. UNIT CAPACITY FACTOR (USING DESIGN MWe)                                       | 27.0       | 8.0          | 59.9        |
| 18. UNIT FORCED OUTAGE FACTOR (%)   | 21.5       | 5.4          | 13.1        |
| 19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH): |            |              |             |
| 20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:               |            |              |             |



TABLE 3  
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373  
UNIT LASALLE TWO  
DATE May 11, 1992  
COMPLETED BY M.J.CIALKOWSKI  
TELEPHONE (815) 357-6761

REPORTING PERIOD: April 1992

| DAY  | POWER | DAY  | POWER |
|------|-------|------|-------|
| ---- | ----- | ---- | ----- |
| 1    | -12   | 17   | -11   |
| 2    | -12   | 18   | -12   |
| 3    | -12   | 19   | -5    |
| 4    | -12   | 20   | 129   |
| 5    | -12   | 21   | 232   |
| 6    | -12   | 22   | 764   |
| 7    | -12   | 23   | 801   |
| 8    | -12   | 24   | 792   |
| 9    | -12   | 25   | 962   |
| 10   | -12   | 26   | 962   |
| 11   | -12   | 27   | 950   |
| 12   | -2    | 28   | 973   |
| 13   | 69    | 29   | 972   |
| 14   | 203   | 30   | 967   |
| 15   | 108   |      |       |
| 16   | -12   |      |       |

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%  
(UNIT 2)

| YEARLY<br>SEQUENTIAL<br>NUMBER | DATE<br>(YYMMDD) | TYPE<br>F: FORCED<br>S: SCHEDULED | DURATION<br>(HOURS) | REASON | METHOD OF<br>SHUTTING DOWN<br>THE REACTOR OR<br>REDUCING POWER | CORRECTIVE<br>ACTIONS/COMMENTS<br>(LER/DVR # if<br>applicable)  |
|--------------------------------|------------------|-----------------------------------|---------------------|--------|--|---|
| 01                             | 920104           | S                                 | 282.12              | C      | 1  | Refueling outage L2R04.   |
| 02                             | 920412           | F                                 | 7.62                | A      | 4  | Manual turbine trip due to high vibrations caused by rubs on the newly installed turbine rotors (reactor remained critical).            |
| 03                             | 920413           | F                                 | 6.92                | A      | 4  | Manual turbine trip due to high vibrations caused by rubs on the newly installed turbine rotors (reactor remained critical).            |
| 04                             | 920415           | F                                 | 86.77               | A      | 2  | Manual reactor scram due to erratic behavior of the main turbine electro hydraulic control system. (LER #92-004-00 -- DVR #1-2-92-0048) |
| 05                             | 920420           | S                                 | 1.55                | B      | 4  | Turbine overspeed trip test (reactor remained critical).  |

## SUMMARY OF OPERATION:

The unit ended a scheduled refueling outage on 4/12/92. Following the return to service the unit experienced three separate manual Main Turbine trips and a manual Reactor scram. The unit was returned to service on 04/20/92 and remained at high power for the remainder of the month.

ZCADTS/7

F. UNIQUE REPORTING REQUIREMENTS (Unit 2)

1. Safety/Relief Valve Operations

(None.)

2. ECCS System Outages

(See Table 5.)

3. Changes to the Off-Site Dose Calculation Manual.

(None.)

4. Major changes to Radioactive Waste Treatment Systems.

(None.)

5. Indications of Failed Fuel Elements.

(None.)

(Unit 2)  
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

| <u>OUTAGE NO.</u> | <u>EQUIPMENT</u>         | <u>PURPOSE</u>                      |
|-------------------|--------------------------|-------------------------------------|
| 1556              | 2E51-C002                | Recouple turbine.                   |
| 1557<br>1624      | 2E51-D316                | Remove blind flange.                |
| 1575              | 2E12-F024B<br>2E12-F027B | Shutdown cooling requirements.      |
| 1602              | 2E12-F009                | Administrative control.             |
| 1605              | 2E22-F038                | Valve inspection.                   |
| 1607              | 2E51-C005                | Pump and motor maintenance.         |
| 1609              | 2E51-F045                | Repair trip linkage.                |
| 1614              | 2E51-C002                | Repair trip/throttle valve.         |
| 1622              | 2E51-C001                | Inspection of manual trip assembly. |
| 1623              | 2E51-C005                | Repair vacuum pump seal.            |
| 1629              | 2E22-C302A               | Inspection.                         |
| 1631              | 2E22-C302B               | Inspection.                         |
| 1637              | 2E12-F024A<br>2E12-F027A | Shutdown cooling requirements.      |
| 1651              | 2E12-F332B               | Inspection                          |
| 1696              | 2E22-S001                | Administrative control.             |