BOSTON EDISON
Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

E. Thomas Boulette, PhD
Vice President Nuclear Operations
and Station Director

May 13, 1992
BECo Ltr. #92-59

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

License No. DPR-35
Docket No. 50-293

Subject: April 1992 Monthly Report

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.

E T Boulette

E. Thomas Boulette

WJM/bal

Attachment

cc: Mr. Thomas T. Martin
Regional Administrator, Region 1
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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-293
UNIT Pilgrim 1
DATE May 13, 1992
COMPLETED BY W. Munro
TELEPHONE (508) 747-8474

MONTH April 1992

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	667
2	0	18	666
3	0	19	666
4	0	20	666
5	0	21	665
6	0	22	665
7	0	23	666
8	0	24	666
9	0	25	667
10	0	26	667
11	0	27	597
12	0	28	649
13	83	29	667
14	223	30	667
15	527	31	N/A
16	655		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-293
DATE May 13, 1992
COMPLETED BY W. Munro
TELEPHONE (508) 747-8474

OPERATING STATUS

Notes

1. Unit Name Pilgrim 1
2. Reporting Period April 1992
3. Licensed Thermal Power (MWt) 1998
4. Nameplate Rating (Gross MWe) 678
5. Design Electrical Rating (Net MWe) 655
6. Maximum Dependable Capacity (Gross MWe) 690
7. Maximum Dependable Capacity (Net MWe) 670
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
   None
9. Power Level To Which Restricted, If Any (Net MWe) None
10. Reasons For Restrictions, If Any N/A

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	719.0	2903.0	169991.0
12. Number Of Hours Reactor Was Critical	471.0	2525.6	100886.2
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	422.9	2473.5	96976.4
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated(MWH)	777744.0	4817736.0	168522216.0
17. Gross Electrical Energy Generated(MWH)	267650.0	1665150.0	56871564.0
18. Net Electrical Energy Generated (MWH)	257485.0	1603258.0	54654206.0
19. Unit Service Factor	58.8	85.2	57.0
20. Unit Availability Factor	58.8	85.2	57.0
21. Unit Capacity Factor (Using MDC Net)	53.5	82.4	48.0
22. Unit Capacity Factor (Using DER Net)	54.7	84.3	49.1
23. Unit Forced Outage Rate	41.2	14.8	12.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
    Midcycle outage - October 1992 Approximately 35 days

25. If Shut Down At End Of Report Period, Estimated Date of Startup N/A

BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

Operational Summary for April 1992

The unit started the reporting period in cold shutdown with outage activities in progress. On April 9 at 0859 hours the reactor was declared critical and power was increased to approximately 15 percent core thermal power (CTP) to perform post work testing (Temporary Procedure TP 92-021) for the Reactor Water Level Temporary Modification TM 92-13. Reactor shutdown commenced on April 10 at 1818 hours with cold shutdown achieved on April 11 at 0335 hours. On April 12 at 2225 hours the Reactor Mode Selector Switch was taken to STARTUP and the reactor was declared critical at 2225 hours. On April 13 at 0904 hours the generator was synchronized to the grid at approximately 20 percent core thermal power (CTP). Power ascension continued, and at approximately 28 percent CTP a backwash of the main condenser was performed. Following the backwash power ascension continued and the unit attained 100 percent CTP on April 16. This power level was maintained until April 27 when at approximately 1800 hours, power was reduced to approximately 50 percent CTP to perform a backwash of the main condenser. The unit was returned to 100 percent CTP on April 28 at approximately 0600 hours and remained at that level for the remainder of the reporting period. Minor power reductions were initiated on April 18 and 25 to perform control rod exercises.

Safety Relief Valve Challenges
Month of April 1992

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during this reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure, auto signal (ADS) or control switch (manual). Ref. BECo ltr. #81-01 dated 01/05/81.

REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.

2. Scheduled date for next refueling shutdown: April 3, 1993

3. Scheduled date for restart following next refueling: June 8, 1993

4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.

5. See #6.

6. The new fuel loaded during the 1991 refueling outage was of the same design as loaded in the previous outage and consisted of 168 assemblies.

7. (a) There are 580 fuel assemblies in the core.

   (b) There are 1489 fuel assemblies in the spent fuel pool.

8. (a) The station is presently licensed to store 2320 spent fuel assemblies. The actual usable spent fuel storage capacity is 2320 fuel assemblies.

   (b) The planned spent fuel storage capacity is 2320 fuel assemblies.

9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 831 fuel assemblies.

Month April 1992

PILGRIM NUCLEAR POWER STATION

MAJOR SAFETY RELATED MAINTENANCE

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
Reactor Core Isolation Cooling (RCIC) System	EGR Actuator	125 volt DC ground alarm received in Control Room.	Under investigation	EGR replaced with new replacement in kind.	To be determined.	None
Reactor Core Isolation Cooling (RCIC) System	Inboard Steam Isolation valve MO-1301-16	MO-1301-16 indicated closed but was actually open.	Valve actuator to yoke cap-screws backed out and the operator became detached from the valve.	Valve actuator was overhauled and re-attached. Valve was diagnostically tested with satisfactory results.	Actuator to yoke cap-screws were torqued to a higher value. Motor operated valve fasteners were either torque checked or visually inspected to verify the fasteners were not loose.	LER 92-003-00
Reactor Core Isolation Cooling (RCIC) System	Outboard Steam Isolation Valve MO-1301-17	Steam leak in Traversing In-core Probe (TIP) room.	Seal ring leakage	MO-1301-17 seal ring replaced and actuator rebuilt. Valve was diagnostically tested with satisfactory results.	N/A	None
Reactor Building Closed Cooling Water (RBCCW) System	Pump P202D	Seal leakage (F&MR 92-84)	Seal wear.	Seal was replaced and P202D was satisfactorily tested using using Procedure 8.5.3.1.	N/A	None

Month April 1992

PILGRIM NUCLEAR POWER STATION

MAJOR SAFETY RELATED MAINTENANCE

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
Control Rod Drive (CRD) System	CRDs 38-23 and 38-31	Leakage under vessel.	Faulty CRD flange 'O' rings	CRDs 38-23 and 38-31 removed, 'O' rings replaced, and CRDs reinstalled. Leak tested satisfactory.	N/A	None
Reactor Protection System (RPS)	Relay 16A-K5A Low water level/ High drywell pressure PCIS relay.	Relay contacts would not energize to cause "B" side reactor building Isolation during Procedure 8.M.2-1.5.8.2 (F&MR 92-72)	Contacts 3&4 dirty.	Contacts were cleaned and Procedure 8.M.2-1.5.8.2 was performed satisfactorly.	N/A	None
Primary Containment Isolation System (PCIS)	Reactor water level transmitters LT-263-58B.	False high reactor water level signals caused Group I isolations of Main Steam isolation valves.	Improper performance of reference leg chamber and equalizing line.	Equalizing line insulation removed to prevent vaporization in line. Tested via Temporary Procedure (TP) 92-21.	Data to be collected weekly to analyze reference leg chamber performance.	LER 92-004-00
Neutron Monitoring System	Intermediate Range Monitor (IRM) "C"	When inserted on SHUTDOWN, IRM "C" remained downscale (F&MR 92-77)	Detector failure	IRM detector replaced and satisfactorily post work tested via Procedure 8.M.1-1.	N/A	None

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-293
NAME Pilgrim 1
DATE May 13, 1992
COMPLETED BY W. Munro
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REPORT MONTH April 1992

NO.	DATE	TYPE[1]	DURATION (HOURS)	REASON[2]	METHOD OF SHUTTING DOWN REACTOR[3]	LICENSE EVENT REPORT #	SYSTEM CODE[4]	COMPONENT CODE[5]	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
03	3/26/92	F	296.1	A	1	92-003-00	BN	ISV	Continuation of forced maintenance outage.

1	2	2	3	4&5
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-1022)