

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Virgil C. Summer Nuclear Station DOCKET NUMBER (2) 0 5 0 0 0 3 9 5 1 OF 0 2 PAGE (3)

TITLE (4) Reactor Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)											
0	7	2	9	8	4	8	4	0	3	2	0	0	0	0								

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)

OPERATING MODE (9) <u>1</u>	20.402(b)	20.406(c)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) <u>0 6 0</u>	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME A. R. Koon, Assoc. Mgr., Reg. Compliance TELEPHONE NUMBER 8 0 3 3 4 5 - 5 2 0 9

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS										
X	J	B	F	C	V	F	1	3	0	Y									

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 29, 1984, at 0459 hours, during a Reactor startup, the Reactor tripped from 60% power due to Lo-Lo Steam Generator Level in "B" Steam Generator. The Lo-Lo level was due to the erratic operation of Main Feedwater Control Valve IFV-488-FW. The erratic operation of the valve was attributed to the deadband adjustment on the control valve's volume booster. The plant responded as expected during the transient. A post trip review was satisfactorily conducted and plans were made for a reactor restart.

Final corrective action taken to prevent recurrence was to adjust the volume booster so that it does not function except during large signal demands.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Virgil C. Summer Nuclear Station	DOCKET NUMBER (2) 0 5 0 0 0 3 9 5 8 4	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 4	0 3 2	0 0 0	2	of 0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

At 0459 hours, July 29, 1984, during a Reactor startup, the Reactor tripped from 60% power due to "B" Steam Generator Lo-Lo Level. After starting an additional feedwater pump, prior to escalating power to 100%, the operator at the controls observed erratic operation of Main Feedwater Control Valve IFV-488-FW and a decreasing water level in "B" Steam Generator. The operator immediately took manual control of IFV-488; however, the plant tripped before manual control could restore steam generator water level. The plant responded as expected during the transient.

The erratic operation of the control valve was attributed to the deadband adjustment on the valve's volume booster, which had been replaced and course adjusted during the recent outage. The deadband on the volume booster was readjusted and was functionally tested. Following a post trip review and subsequent management approval, the reactor was restarted and final adjustments to the valve's volume booster were completed.

Final corrective action taken to prevent recurrence was to further adjust the volume booster so that it does not function except during large signal demands.

SOUTH CAROLINA ELECTRIC & GAS COMPANY

POST OFFICE 784

COLUMBIA, SOUTH CAROLINA 29218

O. W. DIXON, JR.
VICE PRESIDENT
NUCLEAR OPERATIONS

August 28, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

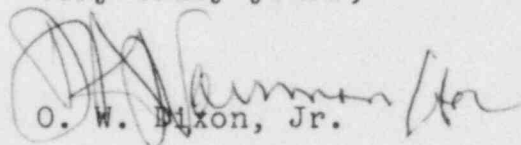
SUBJECT: Virgil C. Summer Nuclear Station
Docket No. 50/395
Operating License No. NPF-12
LER 84-032

Dear Sir:

Please find attached Licensee Event Report #84-032 for the Virgil C. Summer Nuclear Station. This Report is submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv).

Should there be any questions, please call us at your convenience.

Very truly yours,



O. W. Dixon, Jr.

LEK:OWD/lcd
Attachment

cc: V. C. Summer	J. F. Heilman
T. C. Nichols, Jr./O. W. Dixon, Jr.	C. L. Ligon (NSRC)
E. H. Crews, Jr.	K. E. Nodland
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