NAC F	NAC Form 364 (9-43)					LICENSEE EVENT REPORT (LER)					. U.S. W	U.S. NUCLEAR REQULATORY COMMISSION APPROVED OMS NO. 3160-0104 EXPIRES - 9/31/85					
FACILIT	-						01-			44.0	DOCKET NUMBER	The second of the second of	PAGETS				
TITLE I	and the second second	ach	Bott	om Ato	mic Po	wer	Sta	tion	- UI	11t 2	0   6   0   0	10/2//	7 1 OF 0				
		mn T	n10+	Riser	Cafe I	end i	Crac	b-Ti	ko T	ndicat	ione						
	ENT DATE		niec	LER NUMBER			PORT DA		ve T		PACILITIES INVO	LVED (B)					
MONTH	DAY	YEAR	YEAR	SEQUENTING	REVISION NUMBER	MONTH DAY YE		YEAR		FACILITY NA	wes .	DOCKET NUMB	OCKET NUMBERISI				
											0   6   0   0   0   1						
d7	d7 217 81 4 84 - 011 6-			-010	0 0 8 2 78 4				0   5   0   0   0   1								
00	BATI'IG		THIS REP	ORT IS SUBMITTE	D PURSUANT	-		ENTS OF 1	OCFR &: H		of the fallowing) [1	-					
TOWER   01 01 0		20.4 20.4 20.4	20.402(b) 20.406(a)(1)(ii) 20.406(a)(1)(iii) 20.406(a)(1)(iii) 20.406(a)(1)(iv)			20.406(a) 66.38(a)(1) 60.38(a)(2) 60.73(a)(2)(i) 60.73(a)(2)(ii) 60.73(a)(2)(iii)			60.73(a)(2)(v) 60.73(a)(2)(vi) 60.73(a)(2)(vii) 60.73(a)(2)(viii) 60.73(a)(2)(viii) 60.73(a)(2)(viii)		73.71(b) 73.71(c) OTHER (Specify in Abstract bries and in Test, NRC form 366A)						
						ICENSES	CONTACT	FOR THIS	FEW (15)			TELEPHONE NU	MALA				
В	. L.	Cla	rk,	Sr. Eng	ineer	, Spe	ecia	1 Pr	ojec	ts	21 15		-, 5,0 ,1 ,				
				COMPLETE	ONE LINE FOR	EACH CO	MPONEN	FAILURE	DESCRIBE	D IN THIS REPO	AT (13)						
CAUSE	SYSTEM	COMPO	NENT	MANUFAC TURER	PEPORTABLE			CAUSE	SYSTEM	COMPONENT	MANUFAC	REPORTABLE TO NPROS					
Х	A <sub>1</sub> D	R <sub>1</sub> P <sub>1</sub>	Ч	G 0 8 0	Y					111	111						
		1 1	,	111						111	1111						
				SUPPLEME	NTAL REPORT	EXPECTE	D (14)	_			EXPECT	MONT	H GAY YEA				
X 116	6 (11 ym. c)	umpiess (A	PECTEDS	UBMISSION DATE	,		] NO				SUBMISSI DATE II	ON 61 *					
ASTRA	Ab	stra	ct:	2-84-	16			was	shu		See "Cor						

On July 27, 1984, while Unit 2 was shutdown for a refueling and major modification outage, liquid penetrant examinations of the reactor pressure vessel jet pump inlet riser safe ends revealed crack-like indications approximately one inch from the safe end thermal sleeve weld on the jet pump inlet riser safe end. Ultrasonic examination of all the vessel jet pump inlet riser safe ends and the recirculation suction safe ends identified crack-like indications in five of the riser nozzle safe ends. The safe end material is 316 low carbon stainless steel.

A-1

8409050423 840827 PDR ADDCK 05000277 S PDR

1523

NRC form 366A 19.8.31 LICENSEE EVI	U.S. NUCLEAR REGULATORY COMMISSION  APPROVED OMB NO. 3150-0104  EXPIRES. 8/31/86	
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6) PAGE (3)
Peach Bottom Atomic		YEAR SEQUENTIAL REVISION NUMBER NUMBER
Power Station - Unit	2 0 15 10 10 10 1 21 71 7	7 0 1 0 10 0 12 0 0 13

TEXT (If more space is required, use additional NRC Form 368A's) (17)

## Description of the Event:

On July 27, 1984, liquid dye penetrant examination of three of the Peach Bottom Unit 2 jet pump inlet riser safe ends revealed circumferential crack-like indications in the safe end near the thermal sleeve attachment weld in two of the nozzles. These indications were about 0.25 to 0.70 inches in length. A boat sample containing the tip of one of the indications was removed for examination. The safe ends are 12-inch 316 low carbon stainless steel, with a pipe-end wall thickness of 0.83 inches, and a vessel-end wall thickness of 1.20 inches.

Ultrasonic examination of all ten jet pump inlet riser safe ends and both recirculation suction safe ends identified fourteen total circumferential crack-like indications in five of the riser nozzle safe ends. Twelve of the indications are 0.40 inches to 1.25 inches long, one indication is 2.0 inches long and one indication is 3.0 inches long. They are shallow: 24 mils to 72 mils at wall thickness of 0.83 inches; 115 mils to 150 mils at wall thickness of 1.20 inches. One additional axial crack-like indication of undetermined length was also identified in one of these five riser nozzle safe ends. Ultrasonic examinations did not identify any indications in the recirculation suction safe ends.

Independent ultrasonic examination of the jet pump inlet riser safe ends confirmed eight of the crack-like indications (seven circumferential, one axial) in the riser nozzle safe ends.

## Cause of the Event:

Optical microscopic examination of the boat sample indicates that the cracking is intergranular, characteristic of integranular stress corrision cracking.

NRC Form 386A (19-931)  LICENSEE EVENT REF	ENT REPORT (LER) TEXT CONTINUATION					U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMS NO. 3150-0104 EXPIRES: 8/31/85						
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)					PAGE (3)					
Peach Bottom Atomic		YEAR	S	EQUENTIAL NUMBER		REVISION						
Power Station - Unit 2	0  5  0  0  0   2   7   7	84 -	-	1 6		0 10	0   3	OF	0  3			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

## Corrective Actions:

Additional radiographic testing and liquid dye penetrant examinations will be performed on the 316 low carbon stainless steel riser nozzle safe ends. Following these examinations, Philadelphia Electric Company will meet with representatives of the NRC Office of Nuclear Reactor Regulation to discuss the results of these further examinations.

## PHILADELPHIA ELECTRIC COMPANY 2301 MARKET STREET P.O. BOX 8699 PHILADELPHIA, PA. 19101 (215) 841-4000 August 27, 1984 Docket No. 50-277 Document Control Desk U.S. Nuclear Regulatory Commission Washington, DC 20555

Washington, DC 20555

SUBJECT: Licensee Event Report

This LER deals with the discovery of crack-like indications in the safe end - thermal sleeve weld area on the jet pump inlet riser safe end.

Reference:

Docket No. 50-277

Report Number:

2-84-16

Revision Number: 00

2-04-10

Event Date:

July 27, 1984

Report Date:

August 27, 1984

Facility:

Peach Bottom Atomic Power Station RD #1, Box 208, Delta, PA 17314

This LER is submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(i)(B).

Very truly yours,

mulled

W. T. Ullrich Superintendent

Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator Region I, USNRC

Mr. A. R. Blough, Site Inspector

1E22