Omaha Public Power District
444 South 16th Street Mall
Omaha, Nebraska 68102-2247
402/636-2000

January 15, 1996
LIC-96-0004

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, D.C. 20555

Reference: Docket No. 50-285

SUBJECT: December 1995 Monthly Operating Report (MOR)

Enclosed please find the December 1995 MOR for Fort Calhoun Station (FCS) Unit No. 1 as required by FCS Technical Specification 5.9.1.

If you should have any questions, please contact me.

Sincerely,

T. L. Patterson
Division Manager
Nuclear Operations

TLP/dll

Enclosures

c: Winston & Strawn
L. J. Callan, NRC Regional Administrator, Region IV
L. R. Wharton, NRC Project Manager
W. C. Walker, NRC Senior Resident Inspector
R. T. Pearce, Combustion Engineering
R. J. Simon, Westinghouse
INPO Records Center

180071

IE24
1/1

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

DECEMBER 1995
Monthly Operating Report

1. OPERATIONS SUMMARY

During the month of December, 1995, Fort Calhoun Station (FCS) operated at a nominal 100% power with the exception of a one-day power reduction to 99.2% for placing an Ion Exchanger for the Reactor Coolant in service. Normal plant maintenance, surveillance, equipment rotation activities and scheduled on-line modifications were performed during the month. Monitoring of a minor Control Element Drive Mechanism (CEDM) mechanical seal leak continued.

On December 4, 1995, a one hour non-emergency NRC notification was made as a result of the determination that the plant had been outside of its design basis for maintaining an adequate quantity of Trisodium Phosphate (TSP) in the Containment Building to neutralize the sump water to a pH of $\geq$ 7.0. The TSP is stored in the basement of the building and is designed to neutralize the boric acid which would be injected to the Reactor Coolant System (RCS) and containment during a Loss-of-Coolant-Accident (LOCA). The amount of TSP in the FCS containment is sufficient to neutralize the sump water to a pH $\geq$ 7.0 for current boric acid concentrations in the RCS, Safety Injection Tanks, Boric Acid Storage Tanks and the Safety Injection Refueling Water Tank. Corrective actions are being taken as reported in Licensee Event Report (LER) 95-008.

On December 7, 1995, the plant Fire Brigade was alerted and assembled to respond to smoke in the warehouse. The smoke was determined to be caused by an overheated motor on an oscillating fan. No fire suppression system or equipment discharge was required.

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

During the month of December, no Power Operated Relief Valves (PORV) or primary system safety valve challenges or failures occurred.

3. RESULTS OF LEAK RATE TESTS

Although above normal, the December 1995 RCS leak rate was steady at approximately 0.30 gpm throughout the month. This leak rate remained relatively steady following the reactor trip and resultant surveillance testing of the CEDMs on August 26, 1995.

The major contributor to the increase in RCS leakage has been classified as "Known" leakage. This leakage is being collected in the Reactor Coolant Drain Tank (RCDT). The leakage source for "Known" leakage has been attributed to CEDM #15. The "Known" leak rate has decreased slightly over the last several months to approximately 0.20 gpm. The remainder of the leakage has been classified as "Unknown" leakage.

In response to increasing containment activity, a containment entry was made on December 21, 1995 to inspect RCS components for leakage. One or more of the reactor head vent system isolation valves were found to be leaking through. Head vent system valve leakage is currently considered a minor contributor (estimated at less than 0.06 gpm) to both the "Known" and "Unknown" leak rate totals.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No.	Description
172	This amendment revised the Technical Specification on the chemical and volume control system (CVCS) to reformat and clarify the requirements and make them more consistent with the requirements of the Combustion Engineering Standard Technical Specifications (STS) as presented in NUREG-0212, Revision 2.

5. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF DECEMBER 1995

- Refurbished Raw Water Pump Motor AC-10C-M
- Rebuilt Charging Pump CH-1C
- Replaced a broken yoke sleeve on the suction valve for Charging Pump CH-1C
- Replaced the regulator internals on the secondary air start SA-143 for Diesel Generator #1

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE JANUARY 04, 1996
COMPLETED BY D. L. LIPPY
TELEPHONE (402) 533-6843

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: DECEMBER 1995

NOTES

3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
N/A

9. Power Level to which restricted, if any (Net MWe): N/A

10. Reasons for restrictions, if any:
N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period.....	744.0	8760.0	195194.0
12. Number of Hours Reactor was Critical	744.0	7290.1	153708.0
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	744.0	7206.2	151980.5
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1113287.3	10537607.8	202686308.3
17. Gross Elec. Energy Generated (MWH)..	379432.0	3528843.0	66933725.2
18. Net Elec. Energy Generated (MWH)....	362705.4	3365576.5	63857368.8
19. Unit Service Factor.....	100.0	82.3	77.9
20. Unit Availability Factor.....	100.0	82.3	77.9
21. Unit Capacity Factor (using MDC Net)	102.0	80.4	70.7
22. Unit Capacity Factor (using DER Net)	102.0	80.4	69.1
23. Unit Forced Outage Rate.....	.0	3.7	4.0

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
A MAINTENANCE OUTAGE IS SCHEDULED TO OCCUR FROM MARCH 16-23, 1996 TO REPAIR/REPLACE DEGRADING CEDM MECHANICAL SEALS.

25. If shut down at end of report period, estimated date of startup: _____

26. Units in test status (prior to comm. oper.):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY	N/A	
COMMERCIAL OPERATION		

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	JANUARY 04, 1996
COMPLETED BY	D. L. LIPPY
TELEPHONE	(402) 533-6843

MONTH DECEMBER 1995

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	488	17	488
2	488	18	488
3	488	19	487
4	487	20	487
5	488	21	487
6	487	22	487
7	487	23	488
8	487	24	488
9	487	25	488
10	488	26	488
11	488	27	488
12	488	28	487
13	488	29	486
14	488	30	488
15	487	31	487
16	488		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun St.
DATE January 9, 1996
COMPLETED BY D. L. Lippy
TELEPHONE (402) 533-6843

REPORT MONTH December 1995

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
NONE									

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit F - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit H - Same Source

(9/77)

Attachment IV
Refueling Information
Fort Calhoun Station - Unit No. 1

Report for the month ending December 31, 1995

1. Scheduled date for next refueling shutdown.	September 21, 1996
2. Scheduled date for restart following refueling.	November 2, 1996
3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment?	Yes
a. If answer is yes, what, in general, will these be?	Enrichment limit of spent fuel racks is to be increased to at least 4.5 w/o from 4.2 w/o. This is necessary based upon the preliminary Cycle 17 core pattern development.
b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.	N/A
c. If no such review has taken place, when is it scheduled?	N/A
4. Scheduled date(s) for submitting proposed licensing action and support information.	January 1996 (for spent fuel rack enrichment limit change)
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.	N/A
6. The number of fuel assemblies:	
a) in the core	133 Assemblies
b) in the spent fuel pool	618 Assemblies
c) spent fuel pool storage capacity	1083 Assemblies
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.	2007 Outage

Prepared by Ken Holtt Date 1-9-96