Tech. Spec. 6.9.A.2

Boston Edison
Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

L. J. Olivier
Vice President Nuclear Operations
and Station Director

January 12, 1996
BECo Ltr. #96-004

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Docket No. 50-293
License No. DPR-35

DECEMBER 1995 MONTHLY REPORT

In accordance with Pilgrim Nuclear Power Station Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.

L.J. Olivier

RLC/nas/9458

Attachment

cc: Mr. Thomas T. Martin
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector

OPERATING DATA REPORT

DOCKET NO. 50-293
NAME: Pilgrim
DATE: January 12, 1996
COMPLETED BY: R. L. Cannon
TELEPHONE: (508) 830-8321
REPORT MONTH December, 1995

OPERATING STATUS		NOTES
1.	Unit Name	Pilgrim I
2.	Reporting Period	December 1995
3.	Licensed Thermal Power (MWt)	1998
4.	Nameplate Rating (Gross MWe)	678
5.	Design Electrical Rating (Net MWe)	655
6.	Maximum Dependable Capacity (Gross MWe)	696
7.	Maximum Dependable Capacity (Net MWe)	670

8. If Changes Occur in Capacity Ratings (Item Number 3 Through 7) Since Last Report, Give Reasons:

No Changes

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: N/A

		This Month	Yr-to-Date	Cumulative
11.	Hours in Reporting Period	744.0	8760.0	202152.0
12.	Hours Reactor Critical	744.0	7066.0	126266.1
13.	Hours Reactor Reserve Shutdown	0.0	0.0	0.0
14.	Hours Generator On-Line	744.0	6962.8	121823.9
15.	Hours Unit Reserve Shutdown	0.0	0.0	0.0
16.	Gross Thermal Energy Generated(MWH)	1472498.0	13546954.0	216302170.0
17.	Gross Electrical Energy Generated(MWH)	510310.0	4657650.0	73277324.0
18.	Net Electrical Energy Generated(MWH)	492451.0	4485845.0	70443622.0
19.	Unit Service Factor	100.0	79.5	60.3
20.	Unit Availability Factor	100.0	79.5	60.3
21.	Unit Capacity Factor (Using MDC Net)	98.8	76.4	52.0
22.	Unit Capacity Factor (Using DER Net)	101.1	78.2	53.2
23.	Unit Forced Outage Rate	0.0	3.6	12.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each) - NONE
25. If Shutdown at End of Report Period, Estimated Date of Startup - UNIT OPERATING

AVERAGE DAILY UNIT POWER LEVEL

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DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	668	17	668
2	668	18	668
3	668	19	667
4	668	20	667
5	667	21	667
6	668	22	669
7	669	23	669
8	668	24	668
9	667	25	669
10	482	26	668
11	668	27	669
12	668	28	669
13	669	29	668
14	668	30	369
15	661	31	669
16	669		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

OPERATIONAL SUMMARY

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The unit started the reporting period at 100 percent core thermal power (CTP) where it was maintained until 0136 hours on December 10, 1995, when reactor power was reduced to 50% CTP to perform a thermal backwash of the main condenser. Following the backwash, reactor power was increased and the unit obtained 100 percent CTP power at 2320 on December 10, 1995, where it was maintained for the remainder of the reporting period.

SAFETY RELIEF VALVE CHALLENGES

MONTH OF DECEMBER 1995

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during the reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure signal (ADS) or control switch (manual). Reference BECo Ltr. #81-01 dated January 5, 1981.

REFUELING INFORMATION

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The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.

2. Scheduled date for next refueling shutdown: February 1, 1997.

3. Scheduled date for restart following next refueling: March 14, 1997.

4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.

5. See #6.

6. The new fuel loaded during the 1995 refueling outage (RFO-10) is of a different design than that loaded in the previous refueling outage and consists of 136 new fuel assemblies.

7. (a) There are 580 fuel assemblies in the core.

   (b) There are 1765 fuel assemblies in the spent fuel pool.

8. (a) The station is presently licensed to store 3859 spent fuel assemblies. The spent fuel storage capacity is 2891 fuel assemblies. However, 23 spent fuel locations cannot be used due to refuel bridge limitations.

   (b) The planned spent fuel storage capacity is 3859 fuel assemblies.

9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 1103 fuel assemblies.

PILGRIM NUCLEAR POWER STATION MAJOR SAFETY RELATED MAINTENANCE

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SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
Sea Water and Salt Service Water Systems	Monitoring Instrumentation	Traveling screens clogged with seaweed, etc., causing water levels in the sea water and salt service water bays to lower to below normal levels.	The original monitoring system provided minimal monitoring of parameter in the Screenhouse.	Plant Design Change 94-31A implemented to install new instrumentation	New bubbler tubes, new transmitters, and a new panel were installed in the Screenhouse to provide local indication of Cape Code Bay level, Sea Water Bay levels, Salt Service Water Bay levels, and Traveling Screen differential levels. In addition, an Emergency Plant Information Computer (EPIC) graphic was created to display all parameters on EPIC terminals.	None

No major safety related maintenance was completed during this reporting period.

UNIT SHUTDOWNS AND POWER REDUCTIONS

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NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
10	12/10/95	S	0.0	H	N/A	N/A	N/A	N/A	Power Reduction to facilitate a backwash of the main condenser

There were no unit shutdowns during the reporting period.

1	2	3	4&5
F-Forced S-Sched	A-Equip Failure B-Main or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination F-Admin G-Operator Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparations of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)