

LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3180-0104  
EXPIRES - 8/31/85

FACILITY NAME (1) INDIAN POINT UNIT 2 DOCKET NUMBER (2) 050000247 PAGE (3) 1 OF 1

TITLE (4) EXCESSIVE CONTAINMENT ISOLATION VALVE LEAKAGE

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	7	23	84	84	010	0	08	22	84		050000
											050000

OPERATING MODE (8) THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

POWER LEVEL (10) 000	20.402(a)	20.406(a)	50.73(a)(2)(iv)	73.71(a)
	20.406(a)(1)(i)	50.38(a)(1)	50.73(a)(2)(v)	73.71(a)
	20.406(a)(1)(ii)	50.38(a)(2)	50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Test, NRC Form 366A)
	20.406(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
	20.406(a)(1)(iv)	X 50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME MICHAEL BLATT, REGULATORY AFFAIRS TELEPHONE NUMBER 9145265127

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
X	CF		YC4L8	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15) MONTH DAY YEAR

ABSTRACT (Limit to 1600 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 23, 1984 it was determined that a Service Water System Containment Isolation Valve located downstream of the Containment Fan Cooler Units exceeded Technical Specification leakage limits during a periodic surveillance test. The plant was at cold shutdown for a planned refueling and maintenance outage.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  INDIAN POINT UNIT 2	DOCKET NUMBER (2)  0 5 0 0 0 2 4 7 8 4	LER NUMBER (8)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		84	0110	010	2	OF 21

TEXT (17 MORE LINES IS REQUIRED. USE ADDITIONAL NRC Form 366A (17))

While at cold shutdown for a planned outage a scheduled surveillance leakage test of Containment Isolation Valves was in progress. On July 23, 1984 it was observed that Containment Isolation Valve SWN-44-3-A downstream of Containment Fan Cooler #23 exceeded the allowable Technical Specification (4.4.D.2.b) limit. The measured leakage was 0.79 gpm at a test pressure of 52 psig. The allowable leakage rate is 0.36 gpm per fan cooler. The excessive leakage rate was corrected by adjustment of the torque setting on the limitorque operator to achieve tighter seating.

There were no previous similar events.

John D. O'Toole  
Vice President

Consolidated Edison Company of New York, Inc.  
4 Irving Place, New York, NY 10003  
Telephone (212) 460-2533

August 22, 1984

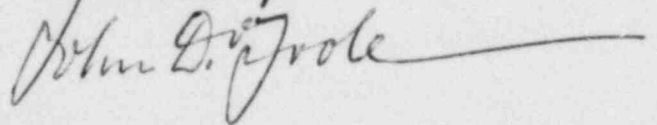
Re: Indian Point Unit No. 2  
Docket No. 50-247  
LER-84-010-00

Document Control Desk  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Dear Sirs:

The attached Licensee Event Report LER-84-010-00 is hereby submitted in accordance with the requirements of 10 CFR Part 50.73.

Very truly yours,



attach.

cc: Dr. Thomas E. Murley,  
Regional Administrator-Region I  
U. S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, Pa. 19406

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U. S. Nuclear Regulatory Commission  
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