#### OYSTER CREEK NUCLEAR GENERATING STATION

PROVISIONAL OPERATING LICENSE NO. DPR-16

TECHNICAL SPECIFICATION CHANGE REQUEST NO. 127 DOCKET NO. 50-219

Applicant submits by this Technical Specification Change Request No. 127 to the Oyster Creek Nuclear Generating Station Technical Specifications, changes to Section 5.3.1.E.

P. B. Fiedler
Vice President and Director
Oyster Creek

Sworn and subscribed to before me this 28th day of August, 1984.

1r/0339e

DIANA A. MALDET
A Notary Public of New Jersey
My Commission Expires June 5, 1986

Diana H. Maldet

#### UNITED STATES OF AMERICA NUCLEAR REGULATORY COMMISSION

IN THE MATTER OF

)

GPU NUCLEAR CORPORATION )

DOCKET NO. 50-219

## CERTIFICATE OF SERVICE

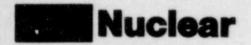
This is to certify that a copy of Technical Specification Change Request No. 127 for the Oyster Creek Nuclear Generating Station Technical Specifications, filed with the United States Nuclear Regulatory Commission on August 28, 1984, has this 28 day of August, 1984, been served on the Mayor of Lacey Township, Ocean County, New Jersey by deposit in the United States mail, addressed as follows:

The Honorable Henry A. Delo, Jr.
Mayor of Lacey Township
818 West Lacey Road
Forked River, N.J. 08731

Vice President and Director Oyster Creek

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**GPU Nuclear Corporation** 

Post Office Box 388
Route 9 South
Forkea River, New Jersey 08731-0388
609 971-4000
Writer's Direct Dial Number:

August 28, 1984

The Honorable Henry A. Delo, Jr. Mayor of Lacey Township 818 West Lacey Road Forked River, NJ 08731

Dear Mayor Delo:

Enclosed herewith is one copy of the Technical Specification Change Request No. 127 for the Oyster Creek Nuclear Generating Station Operating License.

This document was filed with the United States Nuclear Regulatory Commission on August 28, 1984.

Very truly yours,

Vice President and Director

Oyster Creek

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# OYSTER CREEK NUCLEAR GENERATING STATION PROVISIONAL OPERATING LICENSE NO. DPR-16 DOCKET NO. 50-219 TECHNICAL SPECIFICATION CHANGE REQUEST NO. 127

Applicant hereby requests the Commission to change Appendix A to the above captioned license as follows:

## 1. Sections to be changed:

Section 5.3.1.E

## 2. Extent of changes:

To remove the 30 ton limitation on spent fuel shipping casks associated with the spent fuel pool.

## Changes requested:

Replace old pages 5.3-1 with the attached new page 5.3-1.

#### 4. Discussion:

By letter dated March 30, 1977, the NRC issued Amendment No. 22 to the Oyster Creek Technical Specifications. This amendment addressed the increased spent fuel pool storage capacity and the requirements that go with it. One of these requirements had to do with limiting the weight of a spent fuel shipping cask, which could be raised over the top plate of the cask drop protection system (CDPs), to a maximum weight of 30 tons. Although the analysis for the cask drop protection system had been performed by GPUN using a 100 ton cask, and this analysis being found acceptable by the NRC as discussed in their March 30, 1977 SER of Amendment 22, the NRC imposed the 30 ton limitation "until the details of the means used to limit the height to which the cask can be raised over the operating deck have been submitted by the licensee and approved by the NRC staff."

On October 14, 1983, Judge Elfvin of the U.S. District Court, Western District of New York, signed a Partial Settlement Agreement and Order which requires GPUN to return 224 spent fuel assemblies from the Western New York Nuclear Service Center in West Valley, NY to Oyster Creek. This Court Order requires GPUN to use its best efforts to obtain the use of two (2) TN-9 spent fuel shipping casks. Accordingly, in preparation for receiving these fuel assemblies, GPUN is contracting for the use of two (2) TN-9 spent fuel shipping casks each having a full load weight of 40.5 tons. The use of these casks will reduce the number of shipments from West Valley to 32 instead of the 114 required if the NLI 1/2 cask was utilized.

The purpose of this amendment change request is to remove the 30 ton limitation so that GPUN can utilize the TN-9 shipping casks. GPUN is developing and will use a specific procedure for handling the TN-9 casks. To insure that the cask will not be raised more than 6" above the top plate of the (CDPs), limit switches on the crane will be set to limit the cranes upward travel to a level such that the bottom surface of the cask base plate will be limited to a maximum height of six (6) inches above the top plate. In addition, a "GO, NO-GO" gauge will be used to insure the cask is at the correct height prior to movement.

The above requirements that will be incorporated into the procedure for handling the TN-9 shipping casks should satisfactorily resolve the concerns raised in the NRC SER of March 30, 1977, and allow for the removal of the 30 ton limitiation from the Oyster Creek Technical Specification.

## OYSTER CREEK NUCLEAR GENERATING STATION PROVISIONAL OPERATING LICENSE NO. DPR-16 DOCKET NO. 50-219 TECHNICAL SPECIFICATION CHANGE REQUEST NO. 127

Pursuant to 10CFR 50.91, an analysis concerning significant hazards considerations is provided below:

## 1. Section to be changed:

Section 5.3.1.E

## Extent of changes:

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## Changes requested:

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The purpose of this amendment change request is to remove the 30 ton limitation so that GPUN can utilize the TN-9 shipping casks. GPUN is developing and will use a specific procedure for handling the TN-9 casks. To insure that the cask will not be raised more than 6" above the top plate of the CDPs, limit switches on the crane will be set to limit the cranes upward travel to a level such that the bottom surface of the cask base plate will be limited to a maximum height of six (6) inches above the top plate. In addition, a "GO, NO-GO" gauge will be used to insure the cask is at the correct height prior to movement.

The above requirements that will be incorporated into the procedure for handling the TN-9 shipping casks should satisfactorily resolve the concerns raised in the NRC SER of March 30, 1977, and allow for the removal of the 30 ton limitiation from the Oyster Creek Technical Specification.

We have determined that this change request involves no significant hazards considerations in that operation of the Oyster Creek plant in accordance with the proposed amendment will not:

- Involve a significant increase in the probability or consequences of an accident previously evaluated; or
- Create the possibility of a new or different kind of accident from any accident previously evaluated; or
- 3) Involve a significant reduction in a margin of safety.