company South Texas Project Electric Generating Station P. O. Box 289 Wadsworth, Texas 77483 Houston Lighting & Power,

> January 9, 1996 ST-HL-AE-5267 File No.: G26 10CFR50.73

U. S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, DC 20555

The Light

South Texas Project Unit 1 Docket No. STN 50-498 Voluntary Licensee Event Report 95-012 Residual Heat Removal Pump Impeller Cracks due to Improper Manufacturing Processes

South Texas Project submits the attached Unit 1 Voluntary Licensee Event Report 95-012 regarding Residual Heat Removal pump impeller cracks. This event did not have an adverse effect on the health and safety of the public.

If you should have any questions on this matter, please contact Mr. E. D. Halpin at (512) 972-7849 or me at (512) 972-7162.

S. E. Thomas Manager, Design Engineering

DBS/dbs

Attachment: Voluntary LER 95-012 (South Texas, Unit 1)

9601160363 960109 PDR ADOCK

Houston Lighting & Power Company South Texas Project Electric Generating Station

C:

1 1 1 A

Leonard J. Callan Regional Administrator, Region IV U. S. Nuclear Regulatory Commission 611 Ryan Plaza Drive, Suite 400 Arlington, TX 76011-8064

Thomas W. Alexion Project Manager U. S. Nuclear Regulatory Commission Washington, DC 20555-0001 13H15

David P. Loveless Sr. Resident Inspector c/o U. S. Nuclear Regulatory Comm. P. O. Box 910 Bay City, TX 77404-0910

J. R. Newman, Esquire Morgan, Lewis & Bockius 1800 M Street, N.W. Washington, DC 20036-5869

K. J. Fiedler/M. T. Hardt City Public Service P. O. Box 1771 San Antonio, TX 78296

J. C. Lanier/M. B. Lee City of Austin Electric Utility Department 721 Barton Springs Road Austin, TX 78704

Central Power and Light Company ATTN: G. E. Vaughn/C. A. Johnson P. O. Box 289, Mail Code: N5012 Wadsworth, TX 77483 ST-HL-AE-5267 File No.: G26 Page 2

Rufus S. Scott Associate General Counsel Houston Lighting & Power Company P. O. Box 61067 Houston, TX 77208

Institute of Nuclear Power Operations - Records Center 700 Galleria Parkway Atlanta, GA 30339-5957

Dr. Joseph M. Hendrie 50 Bellport Lane Bellport, NY 11713

Richard A. Ratliff Bureau of Radiation Control Texas Department of Health 1100 West 49th Street Austin, TX 78756-3189

U. S. Nuclear Regulatory Comm. Attn: Document Control Desk Washington, D. C. 20555-0001

J. R. Egan, Esquire Egan & Associates, P.C. 2300 N Street, N.W. Washington, D.C. 20037

J. W. Beck Little Harbor Consultants, Inc. 44 Nichols Road Cohassett, MA 02025-1166

									1						
MRC FORM 366 . U.S. NUCLEAR REGULATORY COMMISSION (5-92)							APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95								
(Se	LICENSEE EVENT REPORT (LER)									ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.					
PACILITY NAME (1) South Texas Unit 1								DOCKET	OCKET NUMBER (2) PAGE (3)						
TITLE (4) Residual Heat Removal Pump Impeller Cracks due to								to Improper Manufacturing Processes							
EVE	NT DATE	(5)	1	LER NUMBER (6))	REPOR	RT DAT	8 (7)	OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME			DOCKET NUMBER			
09	26	95	95	012	00	01	09	96	FACILITY NAME			DOCKET NUMBER			
OPER	OPERATING														
NODE (9)			20.40	02(b) 20.405(c)			c)			50.73(a)(2)(iv)			73.71(b)		
POWER LEVEL (10)		100	20.4	20.405(a)(1)(i)			50.36(c)(1) 50.36(c)(2)			50.73(a)(2)(v	()	73.	71(c)		
		100	20.4	20.405(a)(1)(ii)						50.73(a)(2)(vii)		X OTH	ER		
			20.4	20.405(a)(1)(iii)			50.73(a)(2)(1)			50.73(a)(2)(viii)(A)			(Specify in Abstract below and		
				20.405(a)(1)(iv)			50,73(a)(2)(11)			50.73(a)(2)(vili)(B)		IN TEXT, NAC FORM 306A)			
			20.4	20.405(a)(1)(v)			50.73(a)(2)(111)			50.73(a)(2)(x)		VOLUNTARY			
					LICENSEE C	CONTACT P	OR TH	IS LER ((12)						
David Schulker - Consulting Engineering Specialist (512) 972-8517									ta Code)						
			COM	PLETE ONE LINE FO	OR BACH COM	PONENT F	AILURE	DESCRI	BED IN T	HIS REPORT (1	3)				
CAUSE	SYSTE	M	COMPONENT	MANUFACTURER	REPORTABL TO NPRDS	LE S	0	AUSE	SYSTEM	YSTEM COMPONENT MANUFA		TURER REPORTABLE TO NPRDS			
В	BP		P	P025	N										
SUPPLEMENTAL REPORT EXPECTED (14)							And the second second second	and the second second	-	DB/J#RD	MONTH	DA	Y YEAR		
YES	YES(IF yes, complete EXPECTED SUBMISSION DATE).					X	ю		SUI	SUBMISSION DATE (15)					
AD COPPA	ten Itim	- F-0	1400 0000			and the second second	COLUMN STREET, STREET, ST.	a construction of the	ALC: NOT THE REAL PROPERTY OF		the party of the local division of the local		Station in the second state		

On September 26, 1995, Unit 1 was in Mode 1 at 100% power when maintenance personnel, replacing gaskets on Residual Heat Removal (RHR) Pump 1C, noticed cracks on the pump impeller. The RHR pump had been removed from service on September 25, 1995 at approximately 0400 for preplanned maintenance. The impeller was visually inspected and evaluated by South Texas Project Technical Support Engineering. Based on this inspection and discussions with the control room staff, the impeller was removed for additional examination and replaced with a spare on September 27, 1995. On September 28, 1995, the cracked impeller was shipped offsite for examination to determine the cause and mechanism of impeller cracking. RHR Pump 1C was returned to operable status on September 29, 1995 at 0200 hours.

The cause of this event was improper manufacturing of the impeller. Corrective actions include replacement of the RHR Pump 1C cracked impeller with a spare impeller, completion of a detailed engineering evaluation and report, establishing appropriate engineering hold points to ensure quality manufacturing performance for future purchases of spare RHR impellers, and future visual inspections of Emergency Core Cooling System (ECCS) pump impellers as opportunities arise during other corrective/preventative maintenance activities.

NRC FORM 366 (5-92)

NRC FORM 366A	U.S. NUCLE	EAR REGULATORY COMMISSION	APPROVED BY CMB NO. 3150-0104 EXPIRES 5/31/95				
LICE	NSEE EVENT REPORT TEXT CONTINUATION	ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.					
FAC		LER NUMBER (6)	PAGE (3)			
South Texas Unit	1	05000 498	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2.05.4	
south rexas, onit		05000 498	95	012	00	2 OF 4	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

DESCRIPTION OF EVENT:

On September 26, 1995, Unit 1 was in Mode 1 at 100% power when maintenance personnel, replacing gaskets on RHR Pump 1C, noticed cracks on the pump impeller. The RHR pump had been removed from service on September 25, 1995 at approximately 0400 for preplanned maintenance. The cracked impeller was shipped offsite to facilitate a detailed metallurgical examination. Utilizing a spare RHR pump impeller, RHR Pump 1C was returned to operable status on September 29, 1995 at 0200 hours.

The RHR pumps are Model 8X24 SPF single stage pumps manufactured by the Pacific Pumps Division of the Ingersoll-Dresser Pump Company. The pumps were manufactured in accordance with Westinghouse Equipment Specification 952455 as ASME III, Class 2 active pumps. The original RHR pump impellers, including the cracked impeller removed from RHR Pump 1C, were manufactured in 1977-1978 at the Hanford Foundry for Pacific Pumps, Inc. (now owned by Ingersoll-Dresser, Inc.).

The RHR impeller is of the radial, single-suction, closed type design. Water enters the suction eye on the bottom of the pump and is discharged radially to the periphery of the impeller. The impeller is constructed of ASTM A296, Grade CA40, martensitic stainless steel.

Visual and dye penetrant examinations revealed cracks at three different locations: axial cracks on the outside diameter surface of the suction wear ring, cracks adjacent to weld repairs in the impeller vanes, and circumferential cracks in the radius region above the suction wear ring.

Additionally, South Texas Project enlisted industry expertise in the review of analyses and in the impeller cracking root cause determination. While common causal factors exist for all of the identified cracks, different root causes led to crack initiation/propagation.

The axial cracks on the suction wear ring were initiated by stress corrosion cracking. This determination is based on the crack morphology, fractographic examination, and micrographic examination. The root cause of the axial cracks was improper flame hardening of the wear ring which caused the wear ring material to become susceptible to stress corrosion cracking. Specifically, the flame hardening was not applied to sufficient depth and was improperly overlapped. This resulted in increased residual tensile stresses in the material, thereby increasing its susceptibility to stress corrosion cracking.

The cracks adjacent to the vane weld repairs were initiated by stress corrosion cracking. Chemical composition testing of the weld repairs that cracked indicated that the weld filler material was an austenitic wire instead of a martensitic wire as specified in vendor procedures. The use of an austenitic weld filler on a martensitic base material resulted in larger residual tensile stress due to the different coefficients of thermal expansion between the steels. Also, this stress loading would be exacerbated when thermal stresses are applied. The root cause of the weld repair area cracks was the use of improper weld filler material during weld repairs. The use of dissimilar metals in these repairs increased the heat affected zone's susceptibility to stress corrosion cracking.

NRC FORM 3'66A (5-92)		U.S. NUCLEAR REGULATORY COMMISSION NSEE EVENT REPORT (LER) TEXT CONTINUATION			APPROVED BY OMB 10. 3150-0104 EXPIRES F.31/95 ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THI INFORMATION AND RECORDS MANAGEMENT BRANCH (MDB) 7714). U.S. NUCLEAR REGULATORY COMMISSION. WASHINGTON, DC 20555-0001, AND TO THE PAPERWORJ REDUCTION PROJECT (3150-0104), OFFICE OI MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.				
	LICENSEE EVENT								
FACILITY NAME (1) DOCKET NUMBER (2)					LER NUMBER (6) PAGE				
South Taxas Lipit 1			5000 408	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 05 4		
South Texas, Ohn T			03000 498		012	00	5 OF 4		

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

The circumferential cracking above the suction side wear ring was also initiated by stress corrosion cracking. This determination is based on fractographic and micrographic examination of the crack morphology. Overmachining in the radius region of the impeller coupled with flame hardening along the wear ring probably contributed to increased residual tensile stresses in these areas. It is likely that stress corrosion cracking initiated the cracks in this area and also facilitated propagation until the cracks arrested in the thicker vane material. The root cause of the cracks in the radius region was improper machining of the impeller. The residual stresses, in combination with the induced thermal stresses, made the material susceptible to stress corrosion cracking.

CAUSE OF EVENT:

The cause of this event was improper manufacturing of the RHR pump impeller. Inappropriate manufacturing processes included improper flame hardening of the wear ring, incorrect weld material utilized in weld repairs, and over-machining of the radius region.

EVENT ANALYSIS:

On September 25, 1995 at approximately 0400, RHR Pump 1C was removed from service for preplanned maintenance. The allowed outage time for a single train of RHR in accordance with the South Texas Project Technical Specifications is seven days. Utilizing a spare RHR pump impeller, RHR Pump 1C was returned to operable status on September 29, 1995 at 0200 hours, well within the Technical Specification allowed outage time for the RHR pump.

A recognized industry expert was contracted to evaluate the condition of the cracked RHR pump impellers for continued service. This evaluation included stress and fracture mechanics analyses. Potential failure modes were postulated based on the examination of the RHR Pump 1C impeller. These potential failure modes involved the propagation of cracks from areas of over-machining, weld repair, and/or flame hardening. The consequences of the potential failure modes were considered from the standpoint of either catastrophic fracture or excessive distortion. The conclusions are summarized below.

Axial stress corrosion cracking was reported to have initiated in the flame hardened wear ring surface. The dominant service loading of this crack is a design thermal shock. Since the number of thermal transients is small, it was determined that a substantial margin exists against crack propagation by fatigue or fracture. Even if this ring is assumed to be completely fractured, the radial distortion of the ring would not result in interference between the shroud and the stationary wear ring. In short, a through wall axial crack in the impeller wear ring, which propagates the entire axial distance of the wear ring, will not lead to a catastrophic failure of the impeller.

Circumferential stress corrosion cracks were initiated in the vicinity of the minimum wall thickness undercut on the outer diameter of the shroud above the flame hardened wear ring surface. The cracks extended circumferentially and were propagated by stress corrosion cracking resulting from the effect of flame hardening on the adjacent thin section. However, since the pump manufacturer reported that

NRC FORM 366A (5-92)	• •	APPROVED BY OND NO. 3150-0104 EXPIRES 5/31/95							
	LICENSEE EVENT TEXT CONT	ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.							
	FACILITY NAME (1) DOCKET NUMBER (2)					LER NUMBER (6) PAGE (3)			
South Toyon Unit 1		05000 408	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	1 OF 1			
South Texas	s, Unit i		05000 498 95 012		00	4 OF 4			

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

the impeller would be subjected to a pulsating pressure during mid-loop operation, the possibility of fatigue crack growth in thin locations of the ring-to-shroud transition was explored. It was found that fatigue growth would not occur in locations where the minimum wall thickness is $\geq 0.100^{"}$. It was predicted that fatigue cracks could grow in the thinnest of the undercut locations under the pressure pulsations assumed by the manufacturer. However, examination of three other impellers by South Texas Project showed that no other impeller was undercut below 0.175". Fatigue cracks would be deflected from the circumferential direction by intersecting vanes and could subsequently propagate parallel to a vane. These circumferential cracks would arrest in the thicker material along the vanes, where the range of stress intensity decreases below the threshold necessary for crack growth.

Finally, consideration was given as to whether other pumps, including ECCS pumps, might contain impellers in a similar state as found in RHR Pump 1C. The evidence indicates that the cracking identified in the 1C RHR impeller is an isolated condition, and no generic problems exist. This determination is based on a review of the root causes of the failures and the associated supporting analyses. Additionally, the analyses demonstrate that RHR Pump 1C remained capable of performing its intended function, even with the observed cracks in the impeller.

CORRECTIVE ACTIONS:

The following corrective actions have been or will be taken:

- 1. The RHR Pump 1C cracked impeller was replaced with a spare impeller.
- 2. A detailed engineering evaluation was conducted and documented in the Final Engineering Report.
- 3. South Texas Project will take advantage of corrective/preventative maintenance opportunities to perform visual inspections of ECCS pump impellers.
- 4. South Texas Project will establish appropriate engineering hold points to ensure quality manufacturing performance for future purchases of spare RHR impellers.

ADDITIONAL INFORMATION:

None