NÁC Form 19-83)			LICE	LICENSEE EVENT REPORT (LER)						U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85								
ACILITY	NAME (1)	DIO	CANVO	N HNTT 1								OCKET NUMBER (2) PAGE (3) 0   5   0   0   0   2   7  5 1 OF 0   2						
TITLE (4)				N, UNIT 1							0   5   0   0	101	-1/15	1 OF	012			
		ETY	INJEC	TION				101.7					(8)					
EVENT DATE (6) LER NUMBER (6)					REVISION						PACILITIES INVOLVED (8)  NAMES DOCKET NUMBER(S)				-			
MONTH DAY YEAR YEAR NUMBER					NUMBER							0 15 0 10 10 1 1 1						
0 7	2   8	8 4	8 4	-0  2 2	- 010	0   8	2 7	8 4			0   5   0   0   0   1							
	RATING DE (9)	3		ORT IS SUBMITTED	PURSUANT T			ENTS OF 1	CFR §: IC	So.73(a)(2)(iv)	of the following) (1	1	73.71(b)					
POWER		13		102(b) 108(a)(1)(l)		90.36(c			^	60.73(a)(2)(v)		口	73.71(e)					
	LEVEL O. O. O			80.36(c)(2)				50.73(a)(2)(vii)			OTHER (Sp							
20.405(a)(1)(iii) 20.405(a)(1)(iv)				80.73(a)(2)(i) 80.73(a)(2)(ii)				50.73(a)(2)(viii)(A) 50.73(a)(2)(viii)(B)			386A)							
			-	405(a)(1)(iv) 405(a)(1)(v)	H					50.73(a)(2)(x)								
					L	CENSEE	CONTAC	T FOR THIS	LER (12)			7.5	PHONE COLO	950				
NAME		\A\\ -		TCV DEC	LATORY	TORY COMPLIANCE FUCTAGED					AREA CODE	TELEPHONE NUMBER						
DAVID P. SISK, REGULA				LATURY	TORY COMPLIANCE ENGINEER					8,0,5	5,	9 5 1-	17 3	1511				
				COMPLETE	ONE LINE FOR	EACH C	OMPONEN	T AILURE	DESCRIBE	D IN THIS REPO	ORT (13)	_						
CAUSE	SYSTEM	CC -7	ONENT	MANUFAC- TURER	REPORTABLE TO NPRDS			CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER		PORTABLE O NPRDS					
	1	1	11	111					1	111	1111	1						
	1	1	1.1	111	7.7				1	111	1111							
				SUPPLEME	NTAL REPORT	EXPECT	ED (14)				EXPEC		MONTH	DAY	YEAR			
YES (If yes, complete EXPECTED SUBMISSION DATE)						-	X) NO		1		SUBMISE	SUBMISSION DATE (15)			1			
				approximetely fifteen		ewritten la	-				1							
	di re th th re	ue to equine ne Sh ne sp eady ne co rippo team	ned echift fourious state auses ed to General	of this e perform a rator Flow	e of low responde and emer of this event ar a survei w and Pr	d au genc act e as llan	Tave tomat y not uatio	and ticall, tifica on, th lows: est us	high s y. Ar tions e syst the h ing Pr s from	steam flo n Unusual were mad tems were high stea rocedure m Service	ow signal: I Event wa de. Upon e reset an am flow b I-12B1, e". Subs	s. as d rec nd r ista "Rem eque	All leclare cogniti realign ables w noval o	d by on of ed to were				
	To	oola o pro	nt Symetre servent of ro	recurrence om to pro-	rop belo	w 54	abno	(Low-	Low Tatus	avg). board h	as been i	nsta	alled i	n the				
	P	lant	oper	ations.														
	2	035d	/0021	K								1	( ) s					
		84 PI S	10831 OR AL	0143 846 0CK 050	0827 00275 PDR							7	1/					

(9-83)	LICENSEE EVENT REPORT (LER) TEXT CONTINUATION					APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85						
FACILITY NAME (1)		DOCKET NUMBER (2)				LER NUMBER (6)				PAGE (3)		
				YEAR	1	SEQUENTIAL NUMBER		REVISION				
DIABLO CANYON UNIT 1			0 15 10 10 10 1 2 17 1	5 814	_	01212	_	0.10	012	OF	012	

TEXT Iff more space is required, use additional NRC Form 366A's) (17)

At 0709 PDT on July 28, 1984, with the Unit in Mode 3 (Hot Standby), a Safety Injection (SI) (BQ) and Reactor Trip (RT) (JC) occurred, due to a coincidence of low-low Tavg and high steam flow signals. All required equipment responded automatically. An Unusual Event was declared by the Shift Foreman, and emergency notifications were made. Upon recognition of the spurious nature of this actuation, the systems were reset and realigned to ready status.

The causes of this event are as flows: the high steam flow bistables (FFA) (JC) were tripped to perform a surveillance test using Procedure I-12B1, "Removal of Steam Generator Flow and Pressure Channels from Service." After notification that the high steam flow bistables were tripped, steam generator filling continued, causing the average temperature of the Reactor Coolant System (AB) to drop below  $543^{\circ}$  F. This resulted in a Low-Low Tavg condition which, in coincidence with the high steam flow signal, produced the SI and RT. Operations personnel did not sufficiently consider the effect of a low Reactor Coolant System Tavg signal, on the Safety Injection Actuation (JC) coincidence logic.

To prevent recurrence, a plant abnormal status board has been installed in the control room to provide a ready reference of evolutions in progress that could affect plant operations. In addition, the I&C Maintenance Manager has reissued instructions to all I&C technicians requiring that the control room operator be notified immediately before tripping bistales in safety-related systems. This notification is in addition to the regular notification required by procedure.

This event had no safety consequences and in no way affected the health and safety of the public. Had the even occurred at power, this would have been a previously analyzed Condition II event with no effect on the health and safety of the public.

2035d/0021K

## PACIFIC GAS AND ELECTRIC COMPANY

IP G == IE

+

77 BEALE STREET • SAN FRANCISCO, CALIFORNIA 94106• (415) 781-4211 • TWX 910-372-6587

JAMES D. SHIFFER
MANAGER
DEPARTMENT OF NUCLEAR PLANT OPERATIONS
NUCLEAR POWER GENERATION

August 27, 1984

PGandE Letter No.: DCL-84-292

Document Control Desk U. S. Nuclear Regulatory Commission Washington, D.C. 20555

Re: Docket No. 50-275, OL-DPR-76
Diablo Canyon Unit 1
Licensee Event Report 84-022-00
Safety Injection Actuation

## Gentlemen:

Pursuant to 10 CFR 50.73(a)(2)(iv), PGandE is submitting the enclosed Licensee Event Report concerning an actuation of an Engineered Safety Feature (Safety Injection).

This event has in no way affected the public's health and safety.

Sincerely,

Enclosure

cc: J. B. Martin Sarvice List

IE22