



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

December 29, 1995

Mr. D. L. Farrar  
Manager, Nuclear Regulatory Services  
Commonwealth Edison Company  
Executive Towers West II  
1400 Opus Place, Suite 500  
Downers Grove, IL 60515

SUBJECT: INSERVICE TESTING PROGRAM PLAN FOR PUMPS AND VALVES, REVISION 6,  
AND ASSOCIATED REQUESTS FOR RELIEF - BRAIDWOOD STATION, UNITS 1  
AND 2 (TAC NOS. M89332 AND M89333)

Dear Mr. Farrar:

By letters dated December 13, 1993, and March 10, 1994, the Commonwealth Edison Company (ComEd, the licensee) submitted Revision 6 of the Braidwood Inservice Testing Program Plan for Pumps and Valves (IST). Revision 5 of the IST program was approved by letter dated September 14, 1994. The safety evaluation (SE) enclosed with the September 14, 1993 letter, identified eight anomalies in the Braidwood IST program. The licensee responded to these anomalies and requested relief requests VR-2 and VR-28 by letters dated December 13, 1993, and March 10, 1994.

The Nuclear Regulatory Commission (NRC) staff has reviewed and evaluated your response to the anomalies and relief requests VR-2 and VR-28. Based on the information submitted, the staff has determined that request VR-28 conforms with the provisions of Position 2 of Generic Letter (GL) 89-04 and relief is thereby granted. The staff has denied relief request VR-2. However, by letter dated January 31, 1992, the staff determined that Request VR-2 for the Byron Station does conform with GL 89-04, Position 2. Therefore, a similar relief may be used at Braidwood. In order for Braidwood to use a relief similar to the Byron VR-2 relief, Braidwood must comply with the conditions outlined in the Byron SE dated January 31, 1992, which conforms with GL 89-04, Position 2 guidelines on disassembly. The valves must be verified to be at least part-stroke exercised with flow following reassembly prior to returning them to service. The results of the review of your responses to the rest of the anomalies has been summarized in Table 1 of the enclosed SE.

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Mr. D. L. Farrar

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We conclude that the Braidwood Nuclear Power Station, Units 1 and 2, First Ten-Year Interval Inservice Testing Program for Pumps and Valves (Revision 6) with the additional information provided and reliefs granted, or granted with alternative requirements, constitute part of the basis for meeting the requirements of 10 CFR 50.55a(g) and Technical Specification 4.0.5.

Sincerely,

Original signed by  
J. Stang for R. Capra  
Robert A. Capra, Director  
Project Directorate III-2  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-456, STN 50-457

Enclosure: Safety Evaluation

cc w/encl: see next page

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*See Comments*

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Unit Nos. 1 and 2

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