

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Point Beach Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 2 6 6	PAGE (3) 1 OF 0 2
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TITLE (4)
Inadvertent Actuation of Reactor Protection System on Low Pressurizer Pressure

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)
07	21	84	84	004	00	08	20	84	None			0 5 0 0 0
									None			0 5 0 0 0

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 0 0	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(c)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(e)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(e)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)						
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)								

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME C. W. Fay, Vice President-Nuclear Power	AREA CODE 4 1 4	2 7 7	- 2 8 1 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		

SUPPLEMENTAL REPORT EXPECTED (14) <input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While performing maintenance on a turbine first stage pressure instrument, an inadvertent reactor protection actuation occurred. The reactor was shut down with pressurizer pressure below the low pressurizer pressure trip setpoint. When the turbine first stage pressure instrument was placed in test for maintenance purposes, it deenergized interlock circuit P7. This unblocked the low pressurizer pressure trip function and resulted in a reactor protection actuation.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		- 0	0 4	- 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 07/21/84, while performing a maintenance request involving a calibration check of the turbine first stage pressure instruments, an inadvertent actuation of the reactor protection system occurred. The reactor was shut down with the shutdown rods withdrawn and all other rods fully inserted into the core at the time. The primary plant was being maintained at 450 degrees Fahrenheit and 700 psig.

A technician was checking the accuracy of the turbine first stage pressure instruments and their input into the plant computer. This calibration check was being accomplished under a maintenance request and a detailed procedure was not considered to be required. When the first turbine first stage pressure instrument was placed in test, interlock circuit P7 was deenergized. Interlock P7 is an at power trip block that blocks various reactor trip functions at low power levels. It requires three out of four power range channels and two out of two turbine first stage pressure channels below approximately 10 percent power in order to block the associated trip functions. One of the trips blocked by P7 is the low pressurizer pressure trip. When P7 was deenergized with pressurizer pressure below its setpoint, the actuation of reactor protection occurred. All equipment operated as designed.

The primary cause of the reactor protection actuation was the plant conditions required by the maintenance request. The condition stated was, "Any power level," which was sufficient with the plant operating at power. It did not, however, consider the possibility that the maintenance request would be performed with the plant at less than 10 percent power. Subsequently, the maintenance request was performed with the reactor shut down and pressurizer pressure below the trip setpoint resulting in the reactor protection actuation.

The secondary causes of the reactor protection actuation were cognitive errors. The Duty Shift Superintendent did not recognize that the channel was to be placed in test; and the technician did not realize the potential for deenergizing P7 while checking the calibration of the turbine first stage pressure instruments.

This event is being reviewed by operators and supervisors to prevent future occurrences. The maintenance work administrative control procedure is under review and redraft and will be evaluated to upgrade appropriate control of these types of events. No further action is considered to be required.



Wisconsin Electric POWER COMPANY
231 W. MICHIGAN, P.O. BOX 2046, MILWAUKEE, WI 53201

August 20, 1984

Mr. J. G. Keppler, Regional Administrator
Office of Inspection and Enforcement,
Region III
U. S. NUCLEAR REGULATORY COMMISSION
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

DOCKET NO. 50-266
LICENSEE EVENT REPORT NO. 84-004-00
INADVERTENT ACTUATION OF REACTOR PROTECTION SYSTEM
POINT BEACH NUCLEAR PLANT, UNIT 1

Enclosed is Licensee Event Report No. 84-004-00 which provides a description of an inadvertent actuation of the Unit 1 reactor protection system on low pressurizer pressure while in a shutdown condition reportable in accordance with 10 CFR 50.73(a)(2)(iv), "Any event or condition that resulted in manual or automatic actuation of any engineered safety feature, including the reactor protection system."

Very truly yours,

Vice President-Nuclear Power

C. W. Fay
Enclosure

Copy to NRC Resident Inspector

AUG 23 1984

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