NRC Form 300 19-83) L!CENSEE EVENT RE								PORT (LER)		NUCLEAR REGULATORY COMMISSION APPROVED OMS 1.3, 3150-0104 EXPIRES: 8/31/85						
PACILITY	NAME (1	1								0	OCKET NUMBER				£ (3)		
SU	SQUEH	ANNA	STEAM	ELECTRIC	STATIO	N - U	NIT :	2		10	15 10 10	1013	1818	1 OF	0 12		
TITLE IA								17.									
RE	ACTOR	WATI	ER CLE	AN UP													
EVENT DATE (8) LER NUMBER (6)				The second second	REPORT DATE (7)				OTHER FACILITIES INVOLVED (8)								
MONTH DAY		YEAR	YEAR SEQUENTIAL REVEK			MONTH DAY		YEAR	FACILITY NAMES			DOCKET NUMBERISI					
	1					DE A						0 5 0 0 0 1 1					
017	2/3	814	814	-01114	-010	018	2 2	814				0 15	1010	101			
017	-10	-	THIS REP	ORT IS SUBMITTI	D PURSUANT	TO THE R	EQUIREN	ENTS OF 10	CFR 5 10	hack one or more o	t the following) (1	1)					
MODE (9)				20.402(b)			20.406(a)			X 80.73(a)(2)(iv)			73.71(b)				
POWER LEVEL (10) 0 14 14			20.4	106 (a)(1)(d) 106 (a)(1)(E) 106 (a)(1)(III) 106 (a)(1)(IV)	80.38(a)(1) 80.38(a)(2) 80.73(a)(2)(1) 80.73(a)(2)(1) 90.73(a)(2)(1)				80.73(a)(2)(v) \$0.73(a)(2)(vii) \$0.73(a)(2)(viii)(A) \$0.73(a)(2)(viii)(B) \$0.73(a)(2)(x)			73.71(e) OTHER (Specify in Abstract below and in Text, NRC Form 366A)					
						LICENSEE	CONTAC	T FOR THIS	LER (12)								
NAME												TELEPHO	ONE NUME	ER			
	R	. W.	Stanl	ev							AREA CODE						
1											71117	514	121-	1319	13 10		
				COMPLETE	ONE LINE FO	EACH C	OMPONER	T FAILURE	DESCRIBE	D IN THIS REPOR	T (13)						
CAUSE	SYSTEM COMP		ONENT	MANUFAC TURER	MEPORTABLE TO NPROS		CAUSE		SYSTEM	COMPONENT	MANUFAC- TURER		REPORTABLE TO NPROS				
X	KIM	, т	151	X 19 1 9 1	N					1 1 1	1 1 1						
-	TIT	-		AHITL	1	1			1								
		1	1 1	111					111	1.1.1	111						
SUPPLEMENTAL REPORT EXPECTED (14)										MONTH DAY YEA				YEAR			
	S /F ym, c	ramplete l	XPECTED	SURMISS.OF DAT	e)	13	7 NO				SUBMISS DATE	NON		1	1		

At 1759 and 2117 on July 23, 1984, the Reactor Water Clean Up Inlet Out Board Isolation Valve (2F004) closed due to high temperature in the Filter Demineralizer Room. The high temperature detector is a portion of the leak detection system. The high temperature in the Filter Demineralizer Room was caused by problems with the Reactor Building Chilled Water System, which serves as a heat sink for the ventilation system. The "B" Reactor Building Chiller was repaired.

The closing of the Reactor Water Clean Up Inlet Out Board Isolation Valve is an Engineered Safety Features (ESF) actuation since it is a containment isolation.

The event had no adverse effect on the plant.

ASSTRACT (Limit to 1400 speces, i.e., approximately fifteen single-spece hypewritish limits (18)

8408300484 840822 PDR ADDCK 05000388 PDR

> IE22 1/1

LICENSEE EVE		APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85						
FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER (6)		PAGE (3)			
SUSQUEHANNA STEAM ELECTRIC STA	TION	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		T		
UNIT 2						-		

TEXT IN more space is required, use additional NRC Form 386A's) (17)

At 1759 and 2117 on July 23, 1984, the Reactor Water Clean Up Inlet Out Board Isolation Valve (2F004) closed due to high temperature in the Filter Demineralizer Room. The high temperature detector is a portion of the leak detection system. The high temperature in the Filter Demineralizer Room was caused by problems with the Reactor Building Chilled Water System. The "B" chiller would not restart due to a refrigerant problem, the "A" chiller was placed into operation, the temperatures in the Reactor Building Chilled Water System increased slightly prior to settling out. The closing of the Reactor Water Clean Up Inlet Out Board Isolation Valve is an Engineered Safety Features (ESF) actuation since it is a containment isolation.

The low refrigerant temperature cut-out switch was replaced. The new switch was calibrated and tested, and the chiller was run satisfactorily.

Unit 2 was at 44% power at the time of the event. The event had no adverse effect on the plant.

Pennsylvania Power & Light Company

Two North Ninth Street . Allentown, PA 18101 . 215 / 770-5151

August 22, 1984

U.S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION LICENSEE EVENT REPORT 84-014 ER 100450 FILE 841-23 PLA -2288

Docket No. 50-387 / 50-388 License No. NPF-14 / NPF-22

Attached is Licensee Event Report 84-014. This event was determined reportable per 50.73 (a) (2) (iv), in that an unplanned Engineered Safety Festures (ESF) actuation occurred. The event consisted of the Reactor Water Clean Up Inlet Out Board Isolation Valve (2F004) closing due to high temperature in the Filter Demineralizer Room.

H. W. Keiser

2 Keira

Superintendent of Plant-Susquehanna

RWS/jls

cc: Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Mr. R. H. Jacobs Senior Resident Inspector U.S. Nuclear Regulatory Commission P.O. Box 52 Shickshinny, PA 18655

IE22