



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

December 27, 1995

Mr. M. S. Tuckman
Senior Vice President
Nuclear Generation
Duke Power Company
P. O. Box 1006
Charlotte, NC 28201-1006

SUBJECT: SAFETY EVALUATION FOR REVISION 1 TO TOPICAL REPORT DPC-NE-3000-P,
"THERMAL-HYDRAULIC TRANSIENT ANALYSIS METHODOLOGY" MCGUIRE NUCLEAR
STATION, UNITS 1 AND 2; CATAWBA NUCLEAR STATION, UNITS 1 AND 2; AND
OCONEE NUCLEAR STATION UNITS 1, 2, AND 3 (TAC NOS. M90143, M90144,
AND M90145)

Dear Mr. Tuckman:

Your letter of August 9, 1994, submitted a revision to Duke Power Company (DPC) Topical Report DPC-NE-3000-P, "Thermal-Hydraulic Transient Analysis Methodology," dated June 1994, for review. The report describes changes to the DPC thermal-hydraulic transient analysis methodology that are due to: (1) the steam generator replacement for Catawba Unit 1 and McGuire Units 1 and 2, (2) changes to the methodology previously documented in DPC-NE-3000-P, and (3) corrections of typographical errors. Supplemental information was submitted in a letter dated September 12, 1995. This report, for reasons discussed in the enclosed Safety Evaluation, will be referred to hereafter, as Revision 1 to the original DPC-NE-3000-P report, which was issued in its approved form (DPC-NE-3000-PA) by DPC's letter of August 8, 1995.

Since Catawba Unit 2 continues to operate with the currently installed pre-heater steam generators, the previously reviewed and approved steam generator model will continue to be utilized. The DPC-NE-3000-PA report has been augmented in this revision to describe the models to be used for the Catawba Unit 1 and the McGuire Units 1 and 2 new feeding steam generators (FSG).

The staff finds DPC-NE-3000P, Revision 1, to be acceptable for referencing in Catawba, McGuire and Oconee licensing applications to the extent specified, and under the limitations stated, in DPC-NE-3000-P, Revision 1, and the associated NRC Safety Evaluation. The enclosed safety evaluation defines the basis for accepting this Topical Report. The staff was assisted in its review by the International Technical Services (ITS), Inc. The ITS Technical Evaluation Report (TER ITS/NRC/95-4) is also enclosed.

The staff does not intend to repeat its review of the matters described in the Topical Report and found acceptable when the report is referenced in a license application, except to ensure that the material presented is applicable to the specific plant involved. Staff acceptance applies only to the matters described in the Topical Report.

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In accordance with procedures established in NUREG-0390, DPC must publish accepted proprietary and non-proprietary versions of this report. The accepted versions shall incorporate this letter and the enclosed Safety Evaluation between the title page and the abstract. The accepted versions shall include an -A (designating accepted) following the report identification symbol.

Should NRC criteria or regulations change so that staff conclusions regarding the acceptability of the report are invalidated, you will be expected to revise and resubmit your documentation, or to submit justification for continued effective applicability of the Topical Report without revision of the documentation.

This completes NRC actions for TAC Nos. M90143, M90144 and M90145.

Sincerely,

Original signed by:

Robert E. Martin, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270, 50-287
50-369, 50-370, 50-413
and 50-414

Enclosures: (1) Safety Evaluation
(2) Technical Evaluation Report ITS/NRC/95-4

cc w encls: See next page

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Sincerely,



Robert E. Martin, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
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2. Technical Evaluation Report ITS/NRC/95-4

cc w/encls: See next page

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