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April 30, 1992 Fort St. Vrain Unit No. 1 P-92181

Director,
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Docket No. 50-267

SUBJECT: SUPPLEMENT TO APPLICANT'S ENVIRONMENTAL REPORT POST OPERATING LICENSE STAGE

REFERENCES: 1. PSC Letter, Crawford to Weiss, dated July 10, 1991 (P-91219)

- PSC Letter, Warembourg to Weiss, dated March 20, 1992 (P-92123)
- NRC Letter, Erickson to Crawford, dated November 19, 1991 (G-91246)
- PSC Letter, Warembourg to Weiss, dated April 17, 1992 (P-92162)

Gentlemen:

This letter subtast the revised Supplement to Applicant's Environmental Report (ER) - Post Operating License Stage. Public Service Company of Colorado (PSC) submits this ER Supplement to address the environmental impacts of decommissioning the Fort St. Vrain Nuclear Generating Station.

The attached ER Supplement has been updated from the initial submittal (Reference 1), to reflect current decommissioning plans and to incorporate additional information that PSC provided in Reference 2, in response to NRC questions identified in Reference 3 Information from the recently updated Proposed Decommissioning Plan (PDP, Reference 4) has also been incorporated in the attached ER Supplement.

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Changes from the initial ER Supplement (Reference 1) have been marked in the margins on the attachment to facilitate your review. A brief description of these changes is included as Attachment 1. The ER Supplement, included as Attachment 2, has been extensively revised and is intended to completely replace the ER Supplement provided previously.

The attached ER Supplement evaluates the environmental impacts associated with all reasonable decommissioning alternatives. The environmental impacts of decommissioning Fort St. Vrain, by use of the Early Decontamination and Dismantlement (DECON) alternative selected by PSC, are not significant when compared with accepted consequences of operating and decommissioning Light Water Reactor plants. No site specific characteristics were identified that would result in environmental impacts that would be significantly different from those studied generically by the NRC. The attached ER Supplement demonstrates that the impacts of decommissioning the Fort St. Vrain facility do not significantly affect the human environment.

In accordance with 10 CFR 51.55(a), forty-one (41) copies of the attached supplement are included with this letter. Additional copies have been retained by PSC for distribution to other parties or organizations at a later date, if so instructed by the NRC.

If you have any questions regarding the attached information, please contact Mr. M. H. Holmes at (303) 520-1701.

Sincerely,

Donald W. Warembourg

Manager, Nuclear Operations

DWW/SWC/lmg

Attachments

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cc: Regional Administrator, Region IV

Mr. J. B. Baird Senior Resident Inspector Fort St. Vrain

Mr. Robert M. Quillin, Director Radiation Control Division Colorado Department of Health

Dr. Seymour H. Weiss, Director Non-Power Reactor, Decommissioning and Environmental Project Directorate

U. S. Nuclear Regulatory CommissionATTN: Document Control DeskWashington, D.C. 20555

ATTACHMENT 1 TO P-92ERS

REVISIONS TO JULY 1991 SUPPLEMENT TO APPLICANT'S ENVIRONMENTAL REPORT - POST-OPERATING LICENSE STAGE

SECTION 1:

- · Updated to reflect current spent fuel shipping status.
- Updated to reflect recent submittals of Proposed Decommissioning Plan (PDP) and Decommissioning Technical Specifications.

SECTION 2:

- Updated to reflect Activation Analysis Verification efforts, Site Characterization surveys.
- Updated to include tritium leaching information from revised PDP.

SECTION 3:

- Updated schedules to reflect information in recent PDP submittal.
- Added discussion about evaporated tritium vapor being released as gaseous effluent.

SECTION 4:

- Updated tritium release plans consistent with recent PDP submittal and PSC response in Reference 2 (P-92123) to NRC Request for Additional Information (RAI).
- Revised dose projections consistent with revised pathways analysis
 provided in RAI response; dose estimates were further revised from
 those provided in RAI response in P-92123 to incorporate
 additional conservatisms and to enhance the discussion or doses
 resulting from a postulated release of 8000 Curies of tritium.
- Added dose estimate due to inhalation of tritiated water vapor from evaporated PCRV Shield Water.
- Updated Radiation Protection Program description based on revised PDP.
- Identified chemicals for release based on RAI response.

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SECTION 5:

- Revised heavy load drop and added steam generator drop accident analysis, based on revised PDP.
- Deleted discussion of lifetime fatal cancer risk contribution, as accident consequences are adequately analyzed in terms of doses.
- Deleted statement that no in-leakage has been observed in the Reactor Building sump. Water out-leakage from the sump is precluded by the high water table.

SECTION 6:

 Updated description of SAFSTOR and ENTOMB alternatives per RAI response.

SECTION 7:

Incorporated analysis of decommissioning alternatives from RAI response.

SECTIOM 8:

 Added EPA Safe Drinking Water Standard, 40 CFR 141, based on RAI response.

SECTION 9:

Updated document summary to incorporate changes in previous sections.

APPENDIX 1:

 Added new Appendix to address 10 CFR 51.45 (b) considerations, as was provided in RAI response.