

U.S. NUCLEAR REGULATORY COMMISSION
REGION I

Report No. 50-247/84-18

Docket No. 50-247

License No. DPR-26 Priority -- Category C

Licensee: Consolidated Edison Company of New York, Inc.

4 Irving Place

New York, New York 10003

Facility Name: Indian Point, Unit 2

Inspection At: Buchanan, New York

Inspection Conducted: July 17-20, 1984

Inspectors: E. H. Gray for RMC 8/6/84
R. A. McBrearty, Reactor Engineer date

Approved by: J. P. Durr 8/6/84
J. P. Durr, Chief, Materials and Processes Section, EPB, DETP date

Inspection Summary:
Inspection on July 17-20, 1984 (Report No. 50-247/84-18)

Areas Inspected: Routine, unannounced inspection of in-service activities including program review, observations of work in progress, review of NDE procedures and review of licensee actions relative to Regulatory Guide 1.150. The inspection involved 31 hours onsite and 20 hours of in-office review by one regional based inspector.

Results: No violations were identified.

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DETAILS1. Persons ContactedConsolidated Edison Company of New York, Inc. (Con Ed)

- *A. J. Budnick, Manager, Nuclear Power QA
- *J. P. Deane, QA NDE Specialist
- J. Makepeace, Technical Consultant
- F. M. Phillips, Manager, Nuclear Power QC
- J. L. Schwartz, QA Examiner
- *G. Wasilenko, Principal Consultant
- *M. A. Whitney, CRA - Regulatory Affairs

*Denotes those present at the exit meeting on 7/20/84.

2. Inservice Inspection (ISI) Program

The inspector reviewed the following to ascertain compliance with applicable ASME Code Section XI requirements, licensee commitments and regulatory requirements:

- Section 4.2 of the facility Technical Specifications
- Licensee application, dated May 5, 1977, to amend Technical Specification 4.2 in response to NRC requests, dated April 22, 1976 and November 17, 1976.
- Licensee submittals to the NRC updating the ISI program to conform to applicable requirements of ASME Section XI, 1974 edition through Summer 1975 addenda.
- Reactor Vessel Inservice Examination Program Plan, Revision 2.
- QA Procedure No. CI-2401.
- Procedure No. QA-500-2.
- Procedure No. QA-502-2

The first ten year inspection interval at Indian Point Unit 2 will end on August 1, 1984. The examinations which are scheduled during the current refueling outage, including the ultrasonic examination of reactor pressure vessel welds, will complete the applicable ASME Section XI requirements for the interval. These examinations, including the automated ultrasonic examination of reactor vessel welds, are being done by Westinghouse Electric Corporation personnel to the requirements of Section XI, 1974 edition.

The inspector's review and discussions with cognizant licensee representatives indicated that adequate program control is maintained by the licensee and that applicable code and regulatory requirements are being met. The inspector found, additionally, that records are available to ascertain that the required examinations for the interval were completed.

No violations were identified.

3. ISI Procedure Review

The following Westinghouse procedures were reviewed by the inspector to ascertain compliance with applicable ASME code and regulatory requirements, and for technical adequacy:

- Procedure No. ISI-154, Revision 2, "Preservice and Inservice Inspection of Reactor Vessels".
- Procedure No. MRS 2.4.2 GEN-23, "Multi-Frequency Eddy Current Inspection of Heat Exchanger Tubing - Preservice and Inservice".

The above listed procedures describe equipment and techniques and define requirements to perform prescribed examinations and were found to adequately perform their intended function.

No violations were identified.

4. Observations of Work in Progress

The inspector observed the eddy current examination of the following tubes in steam generator No. 24:

<u>Row</u>	<u>Column</u>
7	34
7	33
7	32
7	31
7	30
7	29
7	28
7	27
7	26
7	23
7	22
7	18

The inspector's observations were made to ascertain that the examinations were done in accordance with the governing procedure and that licensee commitments and regulatory requirements were met.

A multi-frequency system was used and data were recorded on magnetic tape and on a strip chart. Westinghouse Electric Corporation personnel performed the examinations from a control trailer outside of the radiation area. Probe location and movement was monitored via television cameras placed at strategic locations in the steam generator and television receivers located in the control trailer.

Eddy current examination data were delivered to an off-site location where Zetec personnel evaluated the results. The inspector observed a portion of the evaluation of the results of the examination of the above listed tubes.

QA personnel at the evaluation station were responsible for assuring that all required examinations and evaluation of each generator tube were completed and documented. The inspector verified that this QA function was being performed.

The licensee plans to examine all of the tubes in steam generators 22 and 24 to the greatest extent possible. The examination was being done from the cold leg side through the U-bend and to the tube sheet on the hot leg side whenever possible. Their plans for examining tubes in the remaining two steam generators were not completed at the time the inspector left the site.

The reactor pressure vessel weld ultrasonic examinations which are scheduled to be completed during the current outage will be done by Westinghouse Electric Corporation personnel using the Westinghouse remote, automatic ultrasonic inspection tool. The inspector observed a portion of the sensitivity calibration check which is done to verify the system calibration. Initial system calibration is done at the Westinghouse Waltz Mill site prior to bringing the equipment to the licensee's facility and must be verified prior to performing any examinations. While performing the verification checks equipment problems were encountered which required removal of the tool from the vessel. No vessel weld examinations were done while the inspector was at the site.

In addition to the above, the inspector reviewed qualification/certification records of Westinghouse and Zetec personnel who participated in eddy current examinations and who were scheduled to perform vessel weld examinations. The inspector's review indicated that the individuals were properly certified in accordance with applicable licensee and regulatory requirements.

No violations were identified.

5. Licensee Compliance With Regulatory Guide (R.G.) 1.150

The inspector reviewed the document entitled "Westinghouse Position on USNRC Regulatory Guide 1.150", dated January 23, 1984, and met with

cognizant licensee personnel to discuss the licensee's actions relative to the R.G.

No formal commitment by the licensee to meet the provisions of the R.G. has been made to the NRC. The licensee's program for ultrasonic examination of reactor pressure vessel welds which come under the purview of the R.G. meets various provisions of the R.G. including equipment calibrations techniques and vessel belt line weld examination.

The inspector found that the licensee's examination meets the intent of R.G. 1.150.

6. Exit Interview

The inspector met with licensee representatives (denoted in paragraph 1) at the conclusion of the inspection on July 20, 1984. The inspector summarized the purpose and the scope of the inspection and the findings.

At no time during this inspection was written material provided by the inspector to the licensee.