

December 28, 1995

Mr. Gary J. Taylor
Vice President, Nuclear Operations
South Carolina Electric & Gas Company
Virgil C. Summer Nuclear Station
Post Office Box 88
Jenkinsville, South Carolina 29065

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION, GENERIC LETTER (GL) 95-03
"CIRCUMFERENTIAL CRACKING OF STEAM GENERATOR TUBES" (TAC NO. M92280)

Dear Mr. Taylor:

The NRC staff issued the subject GL on April 28, 1995. You responded to the GL in June 27 and August 18, 1995 letters. The staff has reviewed your responses and has determined that additional information is required to complete our review.

Pursuant to 10 CFR 50.54(f), you are requested to respond to the enclosed questions within 30 days of receipt. This request is within the original reporting burden for information collection of 350 hours covered by the Office of Management and Budget clearance number 3150-0011, which expires July 31, 1997.

Sincerely,

Original signed by
Stephen Dembek, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-395

Enclosure: Request for Additional Information

cc w/enclosure: See next page

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VIRGIL C. SUMMER NUCLEAR STATION

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Mr. Gary J. Taylor, Vice President
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Request for Additional Information

1. The following areas have been identified as being susceptible to circumferential cracking:
 - a. Expansion transition circumferential cracking
 - b. Small radius U-bend circumferential cracking
 - c. Dented location (including dented TSP) circumferential cracking
 - d. Sleeve joint circumferential cracking

In your response, areas b, c, and d were not specifically addressed although it was indicated that there are several design differences between the steam generators at the plants which have observed circumferential cracking and the V.C. Summer steam generators. Please submit the information requested in Generic Letter (GL) 95-03 per the guidance contained in the GL for these areas (and any other area susceptible to circumferential cracking). The staff realizes that some of the above areas may not have been addressed since they may not be applicable to your plant; however, the staff requests that you clarify this (e.g., no sleeves are installed; therefore, the plant is not susceptible to sleeve joint circumferential cracking).

Please clarify the examinations to be performed during the next steam generator tube inspection outage (i.e., the scope "will meet or exceed those in technical specifications section 4.4.5.3.a") for locations susceptible to circumferential cracking. The response should be in accordance with the guidance contained in GL 95-03 (i.e., scope, expansion criteria, probe type, etc.).

2. During the Maine Yankee outage in July/August 1994, several weaknesses were identified in their eddy current program as detailed in NRC Information Notice 94-88, "Inservice Inspection Deficiencies Result in Severely Degraded Steam Generator Tubes." In Information Notice 94-88, the staff observed that several circumferential indications could be traced back to earlier inspections when the data was reanalyzed using terrain plots. These terrain plots had not been generated as part of the original field analysis for these tubes. For the rotating pancake coil (RPC) examinations performed at your plant at locations susceptible to circumferential cracking during the previous inspection (i.e., previous inspection per your Generic Letter 95-03 response), discuss the extent to which terrain plots were used to analyze the eddy current data. If terrain plots were not routinely used at locations susceptible to circumferential cracking, discuss whether or not the RPC eddy current data has been reanalyzed using terrain mapping of the data. If terrain plots were not routinely used during the outage and your data has not been reanalyzed with terrain mapping of the data, discuss your basis for not reanalyzing your previous RPC data in light of the findings at Maine Yankee.

Discuss whether terrain plots will be used to analyze the RPC eddy current data at locations susceptible to circumferential cracking during your next steam generator tube inspection (i.e., the next inspection per your Generic Letter 95-03 response).