



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 167
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated April 27, 1995, as supplemented November 29, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.167 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 45 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Allen G. Hansen, Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of issuance: December 27, 1995



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 171
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated April 27, 1995, as supplemented November 29, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 171, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 45 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Allen G. Hansen, Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of issuance: December 27, 1995

ATTACHMENT TO LICENSE AMENDMENT NOS. 167 AND 171
TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27
DOCKET NOS. 50-266 AND 50-301

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE

TS Table 15.3.5-1 (page 2 of 2)

TS Table 15.3.5-3 (page 2 of 2)

TS 15.3.5-6

INSERT

TS Table 15.3.5-1 (page 2 of 2)

TS Table 15.3.5-3 (page 2 of 2)

TS 15.3.5-6

TABLE 15.3.5-1 (Continued)
(Page 2 of 2)

<u>NO.</u>	<u>FUNCTIONAL UNIT</u>	<u>CHANNEL</u>	<u>SETTING LIMIT</u>
9.	Degraded Voltage (4.16 KV) (A05, A06)	Disconnection of affected bus from offsite power	≥3937 volts Time delay: <54 seconds without SI signal present. <6.47 seconds with SI signal present.
10.	Loss of Voltage		
	a. 4.16 KV (A05, A06)	Disconnection of affected bus from offsite power Start Diesel	a. ≥3156 volts Time Delay: 0.7-1.0 sec
	b. 480 V (B03, B04)	Load shedding	b. 256 volts ±3% Time delay: ≤0.5 sec

Unit 1 - Amendment No. ~~55, 57, 58, 74, 78, 124, 137~~, 167

Unit 2 - Amendment No. ~~60, 61, 62, 79, 82, 127, 141~~, 171

TABLE 15.3.5-3 (continued)
ENGINEERED SAFETY FEATURES

NO.	FUNCTIONAL UNIT	1 NO. OF CHANNELS	2 NO. OF CHANNELS TO TRIP	3 MINIMUM OPERABLE CHANNELS	4 PERMISSIBLE BYPASS CONDITIONS	OPERATOR ACTION IF CONDITIONS OF COLUMN 3 CANNOT BE MET
b.	Start Turbine-Driven Pump					
i.	Undervoltage on 4KV Buses (A01 & A02)	2/each bus	1/each bus	1/each bus		Be in hot shutdown in 8 hours*
ii.	Low Low Steam Gen. Water Level	3/SG	2/each SG	2/SG**		Be in hot shutdown in 8 hours*
4.	SAFETY-RELATED ELECTRICAL LOADS					
a.	4.16KV Buses (A05, A06)					
i.	Degraded Voltage	3/bus	2/bus	2/bus**		...
ii.	Loss of Voltage	2/bus 3/bus	1/bus 2/bus	1/bus 2/bus**		...
b.	480V Buses (B03, B04)					
i.	Loss of Voltage	3/bus	2/bus	2/bus**		Be in hot shutdown in 8 hours*

* If minimum conditions are not met within 24 hours after reaching hot shutdown, the unit shall be in cold shutdown within 48 hours of the event causing the unit shutdown.

** If a channel is determined to be inoperable, resulting in one less than the total number of channels being operable, power operation may continue if the following conditions are met:

1. The minimum number of operable channels is still satisfied.

2. The affected channels placed in trip within 1 hour.

*** Declare the associated standby emergency power supply inoperable for the affected bus. The applicable Limiting Condition for Operation (LCO) shall be entered. Separate LCOs may be entered for the Degraded Voltage and Loss of Voltage functions.

**** Both switches must be activated simultaneously.

***** Use the 3/bus specification for each A05 and A06 bus that has been modified to the 2 out of 3 logic for the loss of voltage protection function.

in the reactor coolant system and the calculated saturation temperature. As a backup, the Plant Process Computer System (PPCS) displays subcooling margin by both addressable point and on the Safety Assessment System (SAS). A second backup display of subcooling information is available on seismically qualified plasma displays which receive input signals from seismically qualified multiplexing equipment. Control board indications and a saturation curve can be used if failure of all direct subcooling indications occurs.

The 4.16KV degraded voltage setting limit is provided as greater-than-or-equal-to a value with no upper limit. The 4.16KV degraded voltage protection feature is designed to actuate when at least two of the three associated relays operate for the duration of the time delay. The 4.16KV degraded voltage relays are normally set as close as possible to the Technical Specifications setting limit to minimize, to the extent practicable, the possibility of unnecessary actuation of this protection feature.

A degraded voltage condition coincident with a safety injection signal causes the 4.16KV degraded voltage protection function to actuate with a shorter time delay. This prevents starting of engineered safety features, that have safeguards sequence time delays greater than this short time delay, under degraded voltage conditions. The safety injection signal from each unit is provided as an input to the degraded voltage protection for each 4.16KV safeguards bus. The operability requirements for the safety injection protection function are provided in this Technical Specifications section. The safety injection input from a unit to the degraded voltage protection function is only required to be operable when safety injection is required to be operable for that unit. If the safety injection input is found to be inoperable during periods when the safety injection protection function is required to be operable, the applicable actions for inoperability of the 4.16KV degraded voltage protection function must be entered.

Reference

- (1) FSAR - Section 7.5
- (2) FSAR - Section 14.3
- (3) FSAR - Section 14.2.5