

Docket No. 50-423
B14078

Attachment 1

Millstone Nuclear Power Station, Unit No. 3
Proposed Changes to the Operating License

April 1992

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described in the Final Safety Analysis Report, as supplemented and amended;

- (3) NNECO, pursuant to the Act and 10 CFR Parts 30, 40, and 70 to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) NNECO, pursuant to the Act and 10 CFR Parts 30, 40, and 70 to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) NNECO, pursuant to the Act and 10 CFR Parts 30, 40, and 70 to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operations of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provision of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Northeast Nuclear Energy Company is authorized to operate the facility at reactor core power levels not in excess of 3411 megawatts thermal (100 percent rated power) in accordance with the conditions specified herein.

(2) Technical Specifications

The technical specifications contained in Appendix A revised through Amendment No. 63, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Northeast Nuclear Energy Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Fire Protection (Section 9.5.1, SER, SSER 2, SSER 4, SSER 5)

- (a) NNECO shall implement and maintain in effect all provisions of the approved fire protection program as described in its Final Safety Analysis Report for the facility through Amendment 17 and as approved in the SER through Supplement No. 5, subject to provisions (b) and (c) below.
- (b) NNECO may make no change to the approved fire protection program which would significantly decrease the level of fire protection in the plant without prior approval of the

Commission. To make such a change, NNECO must submit an application for a license amendment pursuant to 10 CFR 50.90.

- (c) NNECO may make changes to features of the approved fire protection program which do not significantly decrease the level of fire protection without prior Commission approval provided:
- (i) such changes do not otherwise involve a change in a license condition or Technical Specification or result in an unreviewed safety question (see 10 CFR 50.59); and
 - (ii) such changes do not result in failure to complete the fire protection program approved by the Commission prior to license issuance.

NNECO shall maintain, in an auditable form, a current record of all such changes, including an analysis of the effects of the changes on the fire protection program, and shall make such records available to NRC inspectors upon request. All changes to the approved program shall be reported to the Director of the Office of Nuclear Reactor Regulation, together with the FSAR revisions required by 10 CFR 50.71(e).

(4) Salem ATWS Events Generic Letter 83-28

NNECO shall submit responses to and implement the requirements of Generic Letter 83-28 Salem ATWS Events on a schedule which is consistent with that given in their November 8, 1983 letter as modified by their letters dated March 16, 1984 and September 5, 1985.

- D. Exemptions from certain requirements of Appendix J, 10 CFR Part 50 (Section 6.2.6, SSER 4) and from a portion of the requirements of General Design Criterion 4 (Section 3.9.3.1, SSER 4) of Appendix A to 10 CFR Part 50, have previously been granted. See Safety Evaluation Report Supplement 4, November 1985. With these exemptions the facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rule and regulations of the Commission.
- E. The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Millstone Nuclear Power Station Physical Security Plan," with revisions submitted through March 29, 1988; "Millstone Nuclear Power Station Suitability, Training, and Qualification Plan," with revisions submitted through July 21, 1986; and "Millstone Nuclear Power Station Safeguards Contingency Plan," with revisions submitted through October 30, 1985.

- Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth herein.
- F. Except as otherwise provided in the Technical Specifications or Environmental Protection Plan, Northeast Nuclear Energy Company shall report any violations of the requirements contained in Section 2.C of this license in the following manner: initial notification shall be made within 24 hours to the NRC Operations Center via the Emergency Notification System with written follow-up within thirty days in accordance with the procedures described in 10 CFR 50.73(b), (c), and (e).
 - G. The licensees shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.
 - H. This license is effective as of the date of issuance and shall expire at midnight on November 25, 2025.

FOR THE NUCLEAR REGULATORY COMMISSION
Original Signed by H. R. Denton

Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Attachments/Appendices

1. Appendix A--Technical Specifications (NUREG-1176)
2. Appendix B--Environmental Protection Plan

Date of Issuance: January 31, 1986