



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 25 TO FACILITY OPERATING

LICENSE NO. DPR-22

NORTHERN STATES POWER COMPANY

MONTICELLO NUCLEAR GENERATING PLANT

DOCKET NO. 50-263

1.0 Introduction

By letter dated March 30, 1984, Northern States Power Company (the licensee) proposed changes to the Technical Specifications of Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant. The revisions to the Technical Specifications would extend the allowable interval between integrated containment leakage rate tests, add requirements pertaining to the recently-installed intake structure sprinkler system, and make various non-safety related changes to the Technical Specifications.

2.0 Discussion

The proposed revisions to the Technical Specifications would accomplish the following:

1. Extend the end of the allowable interval between integrated containment leakage rate tests from July 8, 1984 until the end of the 1984 refueling outage, so that outage work need not be interrupted for testing. Restart from the 1984 refueling outage is presently expected to occur during October 1984.
2. Add surveillance requirements and limiting conditions of operation for the recently installed intake structure sprinkler system.
3. Drop Operations Committee review of non-safety related procedures governing work activities exclusively applicable to or performed by the guards.
4. Make various administrative changes, all of which fall in the following categories:
 - a) typographical corrections;
 - b) format changes;
 - c) page number changes;
 - d) wording clarification; or
 - e) provision of additional basis.

3.0 Evaluation

Proposed change no. 1 pertains to the time between primary containment leakage rate tests. Primary containment integrity is required during reactor operation or when the reactor water temperature is above 212°F and fuel is in the reactor vessel. The integrated containment leak rate test which is required to be performed at a test interval of 40 ± 10 months, provides intermittent assurance against degradation of containment integrity. Primary containment integrity is not required for the interval of the test extension.

The Technical Specifications require a test interval of 40 ± 10 months for the overall integrated containment leakage rate test. The last test was completed on May 8, 1980. The next test must be performed by July 8, 1984 to meet this requirement. Due to the length of the current refueling outage, which has been extended to mid-October 1984 to accommodate replacement of the recirculation system piping, the test cannot be completed as required, without performing the test at the middle or beginning of the outage.

Because of the number of plant maintenance and modification projects in progress, including containment modifications as part of the Mark I Containment Long Term Program, the containment integrated leakage rate test should be scheduled at the end of the outage following all major work. This will provide assurance of the integrity of the containment vessel following this period of extensive maintenance and modification. This would require a one-time deviation from the 40 ± 10 month schedule. Performance of this test at the beginning or midway through the outage would serve no practical purpose, would severely impact the outage schedule, and would add to the occupational radiation exposure incurred.

Thus we conclude that the proposed change in the Technical Specifications to extend, on a one-time basis, the containment integrated leak rate test schedule is acceptable.

Proposed change no. 2 adds limiting conditions for operation and surveillance requirements for the intake structure sprinkler system constituting additional limitations, restrictions and controls not presently included in the Technical Specifications. The change was proposed to add to the assurance of operability for the recently-installed intake structure sprinkler system. Because the requirements are consistent with requirements for other similar fire protection equipment, we find this proposed change acceptable.

Proposed change no. 3 involves a requirement incorrectly included in the Technical Specifications which requires that non-safety related procedures be reviewed by the Operations Committee. Because the change involves the correction of an error, the change is administrative, and we find the change acceptable.

Proposed change no. 4 involves no changes in requirements and is purely administrative and is, therefore, acceptable.

4.0 Environmental Considerations

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupation radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 Conclusions

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: V. Rooney

Dated: August 15, 1984