



Crystal River Unit 3 Docket No. 5V-302

> April 27, 1992 3F0492-12

U. S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, D. C. 20555

Subject: Licensee Event Report (LER) 92-02

Dear Sir:

Enclosed is Licensee Event Report (LER) 92-02 which is submitted in accordance with 10 CFR 50.73.

Sincerely,

Tary & Boldt

G. Boldt Vice President Nuclear Production

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Enclosure

xc: Regional Administrator, Region II Project Manager, NRP. Senior Resident Inspector



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CRYSTAL RIVER UNIT 3 (CR-3)		DOCKET NUMBER (2)			1	ER NUMBER	(6)		PAG	ME (3)	
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# EVENT DESCRIPTION

On March 27, 1992, Crystal River Unit 3 was shutdown in MODE III (HOT STANDBY) with a Reactor Coolant System (RCS) temperature of 515 degrees F and a pressure of 2000 psig. A reactor trip had occurred earlier. The plant was being cooled down to approximately 300 degrees in accommodate work on the Position Indicator (PI) system of the Control Rod Drive Mechanisms (CRDM)[AA,ZI]. Vital Bus Inverter/Transformer IC (VBIT-1C) [Et,INVT] was out of service and Technical Sperification 3.8.2.1 Action Statement b applied. VBIT-1C is the normal power supply to the 'C' Vital Bus [EF,BU]. The reactor trip resulted from a loss of power to the Engineered Safeguards (ES) 4160V Busses [JE,BU] which also automatically started and loaded the Emergency Diesel Generators (EDG)[EK,DG]. Prior to the reactor trip, EDG-3B had a one gallon per hour (gph) leak from the jacket coolant pump (DJP-2) [EK,P]. The leakage was being made up regularly. Repair of the leak had been scheduled for the foilowing week.

The Nuclear Shift Supervisor On Duty (NSSOD) was informed by at off-duty Shift Technical Advisor that leamage from the seal of DJP-2 had increased to approximately 2-3 gallons per minute (gpm) with the diesel running and make up to account for the increased leakage was difficult. At this point, the operability of EDG-3B was questioned. The ES Busses were placed on the Offsite Power Transformer (OPT) [EA,XFMR] and EDG-3B was shut down at 1538.

After the diesel was shutdown, the Auxiliary Building Operator reported that the leakage had decreased although the volume of the leak was higher than before the trip. Discussions were held with the Engineer responsible for the EDG system, the On Duty Shift Technical Advisor, and management personnel concerning condition and operability of EDG-3B. The NSSOD contacted the Director of Nuclear Plant Operations and informed him of the situation and that he was declaring EDG-3B inoperable at 2330 on March 27, 1992. The combination of VBIT-1C and EDG-3E being inoperable required that the plant enter Technical Specification 3.0.5. This required a cooldown into MODE V (Cold Snutdown). This event is reportable under 10 CFR 50.73(a)(2)(i)(A) and 10 CFR 50.73(a)(2)(i)(B).

# CAUSE

Investigation revealed the shaft seals for DJP-2 contained elastomer O-rings and seal bellows. These components were original plant equipment and had worn out in service. The original 1 gph leak had been evaluated as not being sufficient to cause an operability problem prior to the repair scheduled for the following week. However, following the automatic start of EDG-3B on less of the OPT, the leakage had increased to the point where makeup for the leak was no longer practical and the Nuclear Shift Supervisor determined the EDG was not OPERABLE. The EDG is manufactured by Colt Industries, model number 38TD8-1/8.

LICENSEE EVENT REPO TEXT CONTINUATION	IB. MARLEAR PERIAATORY COMMISSION	APPROVED OWN NO. 3150-0164 EXPINES 4/30/02 ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THUS INFORMATION COLLECTION RECUEST 50.0 HOLIRS. FORMARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20565, AND TO THE PAPERWORK REDUKTION PROJECT (3150-0164), OFFICE OF MANAGEMENT AND BURDET WASHINGTON DT 20565						
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# EVENT EVALUATION

Imposition of the Action Statement for the inoperable DG resulted in cooling down to less than 200 degrees (COLD SHUTDOWN) and instituting decay heat removal cooling as the method of cooling the core. There were no radioactive releases associated with placing the plant in MODE V. The evolution was accomplished using normal operating procedures. Emergency procedures were not required. Public health and safety was not compromised by the inoperability of the EDG because other power sources were available to power vital equipment. All vital equipment could have been retained in service by placing the 4160V ES Bus 3B on the Startup Transformer [EA,XFMR]. Each piece of equipment affected also has redundant equipment powered from the other EDG (EDG-3A).

# CORRECTIVE ACTION

- 1. The leak was repaired on DJP-2A.
- The Preventative Maintenance (PM) program will be modified to include routine replacement of the elastomer components in the seals of pumps on the EDGs.
- Remaining pumps on EDG-3A and EDG-3B will be evaluated to determine if the elastomer seals require replacement.

#### PRFVIOUS SIMILAR EVENTS

evious similar events have occurred.