

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION RELATED TO AMENDMENT NO. 118 TO FACILITY OPERATING LICENSE NO. NPF-2 AND AMENDMENT NO. 109 TO FACILITY OPERATING LICENSE NO. NPF-8 SOUTHERN NUCLEAR OPERATING COMPANY, INC.

JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-348 AND 50-364

1.0 INTRODUCTION

By letter dated September 26, 1995, the Southern Nuclear Operating Company, Inc., et al. (the licensee), submitted a request for changes to the Joseph M. Farley Nuclear Plant, Units 1 and 2, Technical Specifications (TS). The proposed TS would change the containment air lock door seal leakage rate from no detectable leakage to less than or equal to 0.01 L, when the gap between the door seals is pressurized to greater than or equal to 10 psig for a period of not less than 15 minutes.

2.0 EVALUATION

The licensee, in proposing to change the current acceptance criteria from "no detectable seal leakage by pressure decay when the volume between the door seals is pressurized to greater than or equal to 10 psig for at least 15 minutes" to a leakage rate of "less than or equal to 0.01 L, when the gap between the door seals is pressurized to greater than or equal to 10 psig for a period of not less than 15 minutes," has concluded that no detectable leakage represents an impractical criteria to evaluate door seal performance. The licensee has determined that this change in seal leakage criteria will have no impact on the radiological consequences of the loss-of-coolant accident and the rod ejection accident, for which containment leakage is an assumption, because the plant safety analysis is based on a more conservative assumption that the containment leaks at its design leak rate of 0.15 percent per day for the first 24 hours and 0.075 percent per day thereafter for each of these accidents.

In accordance with the requirements of 10 CFR Part 50, Appendix J, Option A, Section III(D)(b)(iv), the licensee's proposed TS change is needed to provide complete documentation of the containment air lock acceptance criteria values. The overall leakage rate of the containment air lock doors has not been changed and remains at less than or equal to 0.05 L, when tested at a pressure greater than or equal to P. The proposed leakage test for each set of door seals is consistent with the acceptance criteria stated in NUREG-1431, "Standard Technical Specifications - Westinghouse Plants" and comparable reduced pressure leak rates previously reviewed and approved by the staff for other plants. The test pressure and duration (equal to or greater than 10 psig for at least 15 minutes) was previously approved by the staff in Supplement 4 of NUREG-0117, the Farley licensing Safety Evaluation Report.

Based on the above, the staff has concluded that the proposed TS change, which establishes an absolute value for the door seal leakage, constitutes performance of a reasonable test that assures the door seals will perform their design function during accident conditions, and is therefore acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of Alabama official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change the surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (60 FR 56370 dated November 8, 1995). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

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Date: December 8, 1995

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