CCN 92-14055

PHILADELPHIA ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION

R. D. 1, Box 208 DELTA, PA 17314

(717) 456-7014

April 27, 1992

Docket No. 50-277

Document Control Desk U. S. Nuclear Regulatory Commission Washington, DC 20555

> SUBJECT: Licensee Event Report Peach Bottom Atomic Power Station - Unit 2

This LER concerns a plant shutdown required by Technical Specifications due to inoperable Reactor Water Leve! Instrumentation.

Reference: Docket No. 50-277 Report Number: 2-92-005 Revision Number: 00 Event Date: 03/25/92 Report Date: 04/27/92 Facility: Peach Boliom Atomic Power Station RD 1, Box 202, Deita, PA 17314

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(i)(A) and 10 CFR 50.73(a)(2)(i)(B).

Sincerely

Kon Power

JE22,

J. J. Lyash, USNRC Senior Resident Inspector cc: T. T. Martin, USNRC, Region I

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KEN POWERS PLANT MANAGER

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On 3/26/92 at 2345, all Unit 2 Reactor Water Level Instrumentation associated with the 2B Reactor Water Level reference leg condensing chamber was declared inoperable. Review by technical personnel concluded that two small instrument fitting leaks outside containment on the 2B reference leg had caused the instrument: to indicate higher than actual and that the instrument would not trip within their TS values. A plant shutdown was immediately initiated to comply with Technical Specification 3.0.C. The cause of this event was a loss of inventory in the 2B condensing chamber reference leg. Following the shutdown, inspections verified no leakage in the Drywell and that the installed which will monitor the Reactor Water Level condensing chamber temperatures. The design and installed configuration will be evaluated. No actual safety consequences occurred as a result of this event. There was one previous similar LER identified. U.S. NUCLEAR REGULATOR' COMMISSION

APPROVED OMB NO 2160-0104 EXPIRES 4/30/92

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

ESTIMATED BURDEN PER RERPONSE TO COMPLY WTH THIS INFORMATION COLLECTION REGUEST 600 HPS FORWARD COMMENTS PEGARDINU BURDEN ESTIMATE TO THE RECORDS AND 7 EPORTS MARAGEMENT BRANCH (P 530) U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF N. NAGEMENT END BUDGET, WASHINGTON, DC 20503.

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NRC FORM 386A

is submitted to satisfy the requirements of 10 C &

Utions at Time of Discovery

Unit in the RUN mode at 100% of rated thermal reactor (EIIS:RPV) power. There we no systems, structures, or components that were inoperable that compributed to this event.

Jescription of Event

On 3/26/92 at 2119 hours, it was noticed that the level indications from the 2B condensing chamber reference leg had drifted approximately five inches higher than indicated on the 2A reference leg. Steady state level indications from the 2A Reactor Water Level condensing chamber were consistent with the 3A and 3B condensing chamber indications.

At 2345 hours after in investigation and troubleshooting, all Unit 2 Reactor Water Level Instruction (EIIS:LI) a sociated with the 2B Reactor Water Level reference leg condensing chamber EIIS:CDU) was declared inoperable. Review by technice' personnel concluded that leaks in the 2B reference leg had caused the instruments to indicate higher than actual and that the instrument would not trip within their TS values. A plant shutdown was imme liately initiated co comply with TS 3.0.C. Prompt notification was made to the Nuclear Regulatory Commission (NRC) on 3/27/92 at 0012 hours. At 0315 hours. at approximately 30% reactor power, the unit was intentionally scrammed. A Primary Containment Isolation System (PCIS)(EIIS:JM) Group II/III isolation occurred as expected due to Reactor Water level decrease after the scram. Level was immediately restored using a Reactor Feed Pump (EIIS:P). At 0325 hours, the PCIS Group II/III isolation logics were reset and affected systems were restored as necessary. The scram logic was reset at 0335 hours after all Control Rod Drives were verified to be inserted. Notification was again made to the NRC at 0941 hours to provide an update of plant conditions. Hot Shutdown and Cold Shutdown conditions were obtained as required by the TS.

Following the shutdown, inspections of the instrument lines in the Drywell verified no leakage out of the system. Leaks on instrumentation fittings outside the Drywell were repaired and the instrument lines were backfilled.

Following inspection and repairs, a plant startup was commenced on 4/4/92 and power operation was achieved on 4/5/92. Reactor water levels were closely monitored after startup to ensure proper level indications.

NRC Form 306A (6-89)

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NRC FJRM 366A U.1 (6/89)	APPROVED OME NO. 3150-0104 EXPIRES 4/30/92											
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condensing chamber reference legs pro instrumentation. The 2B reference le The condensing chamber is a device th into water to provide a constant refe transmitters.	eg is independent and nat supplies makeup b	redunda v conder	ant to nsing s	team								
In December of 1991 during unit start between the instruments fed from the chambers.	tup, a level deviation 2A and 2B Reactor Wa	n was id ter Leve	dentifi el cond	ed lensing								
Walkdowns of accessible instrument se (EIIS:RK) were performed in December identified only one small leak at a p and the instrument body are connected	while at 170 psig pro point where the instru	essure N	which	line								
As steam production increased and cor level indications returned to accepta to monitor the leak and fix the ident was considered safer to monitor the l which could result in a plant transi	able values. The dec ified leak during the leak than attempt a re	ision wi e next (as then outage.	made It								
During the following three months, the indications were closely monitored by difference was very slight and were w Surveillance Test (ST) 9.1-2X, "Surve acceptable.	/ Station Engineering	. Since	e the in	nt level								
On 2/10/02 the Reaster Operator (14)	annad Heilite A.		di ni	in a de la								

On 3/19/92, the Reactor Operator (Licensed, Utility) identified that the wide range Reactor Water Level indications changed slightly and the 2B indication started to trend higher than the other channel. An investigation by Station Engineering indicated that the wide range level instrumentation connected to the 2B condensing chamber was about two and one half inches higher than other wide and narrow range level indications using the process computer points.

Station Engineering walkdowns of accessible instrument sensing lines and instrument racks while at rated reactor pressure identified one additional leak that was not present in December. Instrument equalizing valve positions (EIIS:V) were physically verified. Reactor level, pressure, and flow changes verified that sensing lines were not clogged or affected dynamically.

NRC FÖRM 366A (683)	US, NUCLEAR REGULATORY COMMISSIO	NN .		À			MB NO. 311		4				
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Station Engineering discussed operability at this time and concluded that the instrumentation was operable. This was based on loop milliamp values obtain during troubleshooting. The data was given to the Nuclear E ineering and Services Jivision (NESD) for evaluation. On 3/23/92, NESD evalued the deviation and provided a maximum deviation value of four and one half inches. This amount of deviation would ensure that all instruments connected to the 2B condensing chamber reference leg would trip within their TS values. This amount of deviation was exceeded on 3/26/92.

Cause of the Event

The cause of this event was a loss of inventory in the 2B condensing chamber reference leg. The loss of inventory was caused by leakage in the system for which the condensation rate could not compensate. A small, long-term inventory loss could have caused an increased volume of noncondensable gases within the chamber. The increased collection of noncondensables in the condensing chamber would reduce the condensation capability. This phenomenon could allow even a small leak to cause the inventory loss observed.

Analysis of the Event

During the period that the level indications from the 28 condensing chamber reference leg deviated by a value in excess of the maximum allowed value, no automatic trip functions were challenged. If reactor level had dropped to the low level trip point, primary containment isolation and core and containment cooling initiation would still have occurred. Since there were no single failures, full initiation of the FCIS and the Emergency Core Cooling Systems (ECCS)(EIIS:BJ,JD,BM,BO) would have been initiated by the instrumentation associated with the 2A condensing chamber reference leg and would have occurred within the TS values. The redundant PCIS and ECCS initiations from the 2B condensing chamber reference leg would still have occurred, but the actuations would not have been in accordance with TS requirements.

Since low reactor water levels did not exist over the period discussed, no actual safety consequences occurred as a result of this event. There could be generic implications associated with this event in that similar piping and condensing chamber configuration designs could be affected. The 2A and 2B condensing chambers installed on Unit 3 are of similar design.

Corrective Actions

Following the shutdown, inspections of the instrument lines in the Drywell verified no leakage out of the system and verified that insulation on the steam line going to the condensing chamber was properly installed. Leaks on instrumentation fittings outside the Drywell were repaired and the instrument lines were backfilled.

NRC PÖRM 366A (6-89)	u	S. NUCLEAR REGULATORY CC	APPROVED OM& NO. 3150-0104 EXPIRES: 4/30/92												
	LICENSEE EVENT REPORT TEXT CONTINUATION		ESTIMATED BURDEN PER RESPONSE TO COMPLY WTH THI INFORMATION COLLECTION REQUEST 50.0 HRS. FORWAR COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORD AND REPORTS MANAGEMENT BRANCH (P.630). U.S. NUCLEA REGULATORY COMMISSION WASHINGTON DC 20665. AND T THE PAPERWORK REQUETION PROJECT (J150-0104). OFFIC OF MANAGEMENT AND BUDGET, WASHINGTON DC 20603.										RD IOS IAR TO		
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A Temporary Plant Alteration (TPA) has been installed which will monitor the 2A, 2B, and 3B Reactor Water Level condensing chamber temperatures and ambient temperatures near the 2B condensing chamber. Based on the condensing chamber temperature data obtained from this TPA, the design and installed configuration of the condensing chambers, along with the generic implications associated with this event, will be evaluated. Additional corrective actions or causes resulting from the evaluation will be reported in a revision to this LER as necessary.

Previous Similar Events

There was one previous similar LER (2-90-021) identified in which a shutdown was required due to level instrumentation problems. The corrective actions taken as a result of the previous LER involved tightening the acceptance criteria on the daily ST and increased monitoring of this data. The current event was identified as a result of these corrective actions. As soon as the deviation between redundant instruments exceeded the technical specification limit, a shutdown was initiated.