A6 Inverteus Center Pariway Post Office Box 1295

Telephone 205 877-7122

C. K. McCoy Vice President Nuclea Vogile Preset



April 27, 1992

ELV-03663 000337

1622

Docket No. 50-425

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk. Washington, D. C. 20555

Gentlemen:

VOGTLE ELECTRIC GENERATING PLANT LICENSEE EVENT REPORT RHR VALVE INTERLOCKS OUT OF CALIBRATION DUE TO PROCEDURE INADEQUACY

In accordance with 10 CFR 50.73, Georgia Power Company (GPC) hereby submits the enclosed report related to an event which was discovered on April 2, 1992.

Sincerely,

C.K. MCLUY

CKM/NJS

Enclosure: LER 50-425/1992-003

xc: Georgia Power Company Mr. W. B. Shipman Mr. M. Sheibani NORMS

010010

9205010014 920427 PDR ADDCK 05000425

U. S. Nuclear Regulatory Commission Mr. S. D. Ebneter, Regional Administrator Mr. D. S. Hood, Licensing Project Manager, NRR Mr. B. R. Bonser, Senior Resident Inspector, Vogtle

U.S. NUCLEAR REQULATORY COMPLEXION (5-89) LICENSEE EVENT REPORT (LER)									APPROVED ONB NO. 3150-0104 EXPIRES: 4/30/92							
FACTLETTY NAME (1)	VOG	TLE ELECT	TRIC GEN	RATIN	g plai	rr - u	NIT 2	1	OCKET NUMBE	R (2) 2 5	PAGE 1 OF	<u>73</u>				
RHR VALVE INTER	LOCKS O	UT OF CAL	LIBRATIO	N DUE	TO PRO	CEDURI	E INADI	EQUACY		es les plus la ces	1 1071	-				
EVENT DATE (5)	LI	ER NUMBER	(6)	REPO	RT DAT	E (7)		OTHER	FACILITIES	INVOLVED	(8)					
NONTH DAY YEAR	YEAR	SEQ NUM	REV	MONTH	DAY	YEAR	ł	ACILITY NAM	IE S	DOCKET NUMBER(S) 0 5 0 0 0						
040292	9 2	003	0.0	04	27	92			entre de la contra a fair can deserver	05000						
OPERATING	THIS REP	ORT IS SU	BMITTED P	URSUAN	T TO T	HE REQU	IREMEN	IS OF 10 CF	8 (11)							
MODE (9)	20.40	2(b)		20.405(c)				50.73(a)(2)(iv)	73.71(b)						
POWER	20.40	5(a)(1)(i)	50.36(c)(1) 50.36(c)(2)				50.73(8)(2)(v)	73.71(c)						
LEVEL	20.40	15(a)(1)(i	1).					50.73(8)(2)(vii)	OTHER (Specify in						
	20.40	15(a)(1)(i	(i) X	50.73	(8)(2)	(1)		50.73(a)(2	(VIII)(A)	Abstra	ct below)	£.				
	20.40	5(a)(1)(i	V3	50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)							
	20.40	5(a)(1)(v	1100000	50.73	(8)(2)	(iii)	100 11	50.73(a)(2)(x)							
THE			LICENSE	E CONT	ACT TU	K INIS	LEK (1	6) 101900, 10100000000000000000000000000000		LEBNONE N	HEFE					
AME									31	LEFRONE N	UMBER					
MPUT CURTEANT	AN ACT PAD	CATERINA I	AND CONTRACTOR						LOU LAN	006 000	<u>6</u>					
MERUI SMEIDANI,	NUULEASK	OMPLETE ON	AND CAMPI	DR FACE	FAIL	IPE DES	CRIBED	IN THIS PEP	404 ORT (13)	020-320	9	-				
······		ANTIE AP .	DEDOD7	1		1			L HANDEAG	Lacasar	1					
CAUSE SYSTEM COMPO	NENT	URER	TO NPRDS			CAUSE	SYSTEM	COMPONENT	TURER	TO NPRDS						
		antennintei oremiini				-	-		and the second designed in the second second second		and the second sec					
			1.00													
												-				
	·		1.1	l							l					
	SI	UPPLEMENT	AL REPORT	EXPECT	TED (14		d	time and a state of the state of the state of the		MON	THI DAY T	YEAR				
TYES(If yes, co	nplete E)	XPECTED SU	JBMISTION	DATE)	X	NO		ann a' chuirean an ann an	SUBMISSION DATE (15)	N						
ABSTRACT (16)	No. of Concession, Name of Street, or other	a marka san cake ng mengangan				Accession of the local diversion of the	ant and a fair fair a street		Association and a second	an original descent	andersenada					

On April 2, 1992, with the unit defueled, it was discovered that Unit 2 had operated in a condition prohibited by the Technical Specifications (TS). A procedure writer had found that the procedures used for calibrating the open permissive interlock bistables for the residual heat removal system (RHRS) suction isolation valves specified voltage values which could allow the RHRS suction isolation valves to be opened with reactor coolant system pressure slightly above the TS required maximum of 377 psig. A review of the interlock calibrations on April 2, 1992 found a Unit 2 bistable that had been calibrated in February 1989 with values which established the open permissive interlock for valve 2HV-8702A at approximately 380 psig. However, since this calibration setpoint had been lowered during a subsequent calibration, there was no effect on unit operations.

The cause of this event was incorrect calibration procedures which caused the upper end of the range of acceptable setpoints for the open permissive interlock for valve 2HV-8702A to be higher than the maximum value specified by the TS. The appropriate procedures have been revised.

(6-89) LICENSEE EVENT REPORT TEXT CONTINUATION	CLEAR REGULATORY COMPLESSION	APPROVED CN-8 NO 3150-0104 EXPIRES: 4/30/92									
FACILITY NAME (1)	DOCKET NUMBER (2)	L	LER NUMBER (5)					PAGE (3)			
		YEAR		SEQ N			REV	EV			
VOGTLE ELECTRIC GENERATING PLANT - UNIT 2	05000425	9.2		0	0 3		0	0	2	0F	3

A. REQUIREMENT FOR REPORT

This report is required per 10 CFR 50.73 (a)(2)(i) because the unit had operated in a condition prohibited by the Technical Specifications (TS). A residual heat removal system (RHRS) suction isolation valve was found to have had its open permissive interlock incorrectly set, placing the interlock outside of TS requirements.

B. UNIT STATUS AT TIME OF EVENT

At the time of the discovery of this event, Unit 2 was defueled at 0 percent of rated thermal power. Other than that described herein, there was no inoper the equipment which contributed to the occurrence of this event.

C. DESCRIPTION OF EVENT

On March 25, 1992, a procedure writer was preparing revisions to procedures used to calibrate the RHRS suction isolation valve open permissive interlocks, due to TS changes which revised the open permissive setpoint from 377 to 365 psig. These interlocks ensure that the suction isolation valves between the RHRS and the reactor coolant system (RCS) cannot be opened until RCS pressure drops below 377 psig, as required by TS 4.5.2.d.1.a. The procedure writer found that the procedures called for calibrating the bistables with voltage values which were high enough to allow the valves to open with RCS pressure above 377 psig. After review of existing calibration data, it was found that on two occasions, the as-left voltage values would have allowed the valves to be opened with RCS pressure at approximately 379 psig. However, when the static head pressure of 4.77 psi was taken into account, the calibrated setpoint was found to be less than 377 psig.

On April 2, 1992, further review of the previous interlock calibration data was performed and it was found that a Unit 2 bistable had been calibrated in February 1989 with a value which would have allowed value 2HV-8702A to be opened with RCS pressure at approximately 380 psig, even when taking static head pressure into account. However, there was no effect on unit operations since this setpoint had been lowered during a subsequent calibration. No other incorrect calibrations were found.

D. CAUSE OF EVENT

The cause of this event was incorrect calibration procedures which caused the open permissive interlock setpoint for valve 2HV-6702A to be higher than the maximum value specified by the TS. A review of the procedures and instrument scaling sheets used to prepare the procedures revealed that the procedure writer applied a tolerance band above the expected setpoint. In this case, the upper end of the band for the setpoint tolerance should have prohibited calibration values corresponding to pressures greater than 377 sig.

NRC Form 3 (6-89)	366A *	U.S. NOCLEAR REGULATION COMMISSION LICENS EF. EVENT REPORT (LER) TEXT CONTINUATION							APPROVED ONB NO 3150-010A EXPIRES: 4/30/92									
FACILITY NAME (1)		And Designation States, And Designation of the	p	DOCKET N	NUMBE	R (2)	and a second	LER NUMBER (5)						PAGE (3)				
					100				1	Y SAR	1	SEQ	NUM	T	REV		1	
VOGTL	I AECTRIC	GENERATING	PLANT -	UNIT 2		050	0.0	425		92		0	03		00	3	OF	3

E ANALYSIS OF EVENT

Although the highest found "as-left" calibrated setpoint of approximately 380 paig for the interlock was above the TS limit, it was still well below the design pressure of 600 psig for the RNRS. Additionally, in Settember 1990 "a "as-found" setpoint for this same interlock was bund to be at the same coximate 330-psig value, as in February 1989, including there had been no crument drift. Furthermore, no event has occurred to challenge the system ce administrative controls were in place to prevent operators from opening valves until RCS pressure was less than 365 psig. Based on these considerations, there was no adverse effect on plant safety or the health and mafety of the public as a result of this condition.

F. ORRECTIVE ACTIONS

1. A "Temporary Change to Procedure" (TCP) was initiated for all eight procedures (four each in Unit 1 and Unit 2) used for the interlock calibratic. This prevented future calibrations from being performed using the improper instrument settings. Due to a design change in progress which lowers the TS open permissive setiont value to less than 365 psig in Unit 2, the four Unit 2 procedures were again changed to further lower the instrument settings. The corresponding design charge for Unit 1 is planned for implementation during the Spring 1993 refueling outage

2. An additional review of TS surveillance procedures is being conducted to identify similar misapplica ions of setpoint tolerances. This review will be completed by June 1, 1992, and any additional discrepancies discovered will be reported in a revision to this LER, as necessary.

G. ADDITIONAL INFORMATION

- Failed Components: None
- 2. Previous Similar Events: None
- 3. Energy Industry Identification System Code: Reactor Coolant System - AB Residual Hes* Removal System - BO