



Tennessee Valley Authority Post Office Box 2000, Soddy-Daisy, Tennessee 37379

J. L. Wilson  
Vice President, Sequoyah Nuclear Plant

April 27, 1992

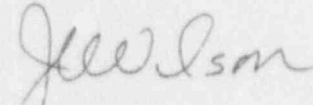
U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 2 - DOCKET  
NO. 50-328 - FACILITY OPERATING LICENSE DPR-79 - LICENSEE EVENT REPORT  
(LER) 50-328/92002

The enclosed LER provides details concerning an inoperable mechanical  
snubber. This event is being reported in accordance with  
10 CFR 50.73(a)(2)(i) as a condition that resulted in an operation  
prohibited by plant technical specifications.

Sincerely,

  
J. L. Wilson

Enclosure  
cc: See page 2

800050  
9204300251 920427  
PDR ADOCK 05000328  
S PDR

*Handwritten initials/signature*

U.S. Nuclear Regulatory Commission

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April 27, 1992

cc (Enclosure):

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LICENSEE EVENT REPORT (LER)

FACILITY NAME\*(1) Sequoyah Nuclear Plant, Unit 2 DOCKET NUMBER (2) | PAGE (3)  
050003 28 10F 04

TITLE (4)  
Inoperable Mechanical Snubber

| EVENT DAY (5) |     |      | LER NUMBER (6)    |                 |       | REPORT DATE (7) |      |                | OTHER FACILITIES INVOLVED (8) |   |   |   |   |   |   |   |   |        |    |     |    |
|---------------|-----|------|-------------------|-----------------|-------|-----------------|------|----------------|-------------------------------|---|---|---|---|---|---|---|---|--------|----|-----|----|
| MONTH         | DAY | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY             | YEAR | FACILITY NAMES | DOCKET NUMBER(S)              |   |   |   |   |   |   |   |   |        |    |     |    |
| 0             | 3   | 2    | 7                 | 9               | 2     | 9               | 2    | 0              | 0                             | 3 | 0 | 0 | 4 | 2 | 7 | 9 | 2 | 050003 | 28 | 10F | 04 |

OPERATING MODE (9) | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following)(11)

|                  |           |           |                 |                   |                   |                     |  |
|------------------|-----------|-----------|-----------------|-------------------|-------------------|---------------------|--|
| 6                | 20.402(b) | 20.405(c) | 50.73(a)(2)(iv) | 73.71(b)          |                   |                     |  |
| POWER LEVEL (10) | 0         | 0         | 0               | 20.405(a)(1)(i)   | 50.36(c)(1)       | 50.73(a)(2)(v)      | 73.71(c)   |
|                  | 0         | 0         | 0               | 20.405(a)(1)(ii)  | 50.36(c)(2)       | 50.73(a)(2)(vi)     | OTHER (Specify in Abstract below and in Text, NRC Form 366A) |
|                  |           |           |                 | 20.405(a)(1)(iii) | XX 50.73(a)(2)(i) | 50.73(a)(2)(vii)(A) |  |
|                  |           |           |                 | 20.405(a)(1)(iv)  | 50.73(a)(2)(ii)   | 50.73(a)(2)(vii)(B) |  |
|                  |           |           |                 | 20.405(a)(1)(v)   | 50.73(a)(2)(iii)  | 50.73(a)(2)(x)      |  |

LICENSEE CONTACT FOR THIS LER (12)

| NAME                                   | TELEPHONE NUMBER      |
|--|-----------------------|
| Jan Bajraszewski, Compliance Licensing | 6 1 5 8 4 3 - 7 7 4 9 |

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS |
|-------|--------|-----------|--------------|---------------------|-------|--------|-----------|--------------|---------------------|
|       |        |           |              |                     |       |        |           |              |                     |

SUPPLEMENTAL REPORT EXPECTED (14) | EXPECTED SUBMISSION DATE (15) | MONTH | DAY | YEAR

X | YES (If yes, complete EXPECTED SUBMISSION DATE) | NO | DATE (15) | 0 | 5 | 2 | 9 | 9 | 2

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On March 27, 1992, at approximately 0750 Eastern standard time, with Unit 2 in a refueling outage (Cycle 5), a snubber was observed to be disconnected from the pipe clamp, and various support attachment bolts were found loose or missing. This condition was detected during performance of a surveillance instruction on an adjacent snubber. Investigation has concluded that the snubber was in the inoperable condition since the previous Unit 2 refueling outage (Cycle 4), at which time the snubber was disconnected to facilitate reactor coolant pump No. 3 main flange retensioning. The cause for why the snubber was not properly reassembled has not yet been determined; discrepancies that have been identified in the work document are under continuing investigation. An engineering analysis was performed demonstrating that plant safety and operability were not adversely affected by the condition. A work request was initiated and completed, correcting the condition.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

| FACILITY NAME (1)             | DOCKET NUMBER (2) | LER NUMBER (6) |          |        | PAGE (3) |   |   |
|-------------------------------|-------------------|----------------|----------|--------|----------|---|---|
|                               |                   | SEQUENTIAL     | REVISION |        |          |   |   |
|                               |                   | YEAR           | NUMBER   | NUMBER |          |   |   |
| Sequoyah Nuclear Plant Unit 1 | 013128            | 92             | 03       | 0      | 0        | 2 | 4 |

TEXT (If more space is required, use additional NRC Form 366A's) (17)

I. PLANT CONDITIONS

Unit 2 was in Mode 6, in the process of refueling.

II. DESCRIPTION OF EVENT

A. Event

At approximately 0750 Eastern standard time (EST) on March 27, 1992, with Unit 2 in a refueling outage (Cycle 5), a snubber (EIIS Code CNB) was observed to be disconnected from the pipe clamp, and various support attachment bolts were loose or missing. Through investigation of Cycle 5 and Cycle 4 work activities, it was determined that the snubber had been in the inoperable condition since the previous Unit 2 refueling outage (Cycle 4), at which time the snubber was disconnected to facilitate reactor coolant pump No. 3 main flange retensioning.

B. Inoperable Structures, Components, or Systems That Contributed to the Event

None.

C. Dates and Approximate Times of Major Occurrences

|                               |   |
|-------------------------------|---|
| September 19, 1990            | The snubber was disassembled to facilitate reactor coolant pump (RCP) main flange retensioning.   |
| October 14, 1990              | A work document indicates that the snubber was reinstalled.   |
| March 13, 1992                | Unit 2 Cycle 5 refueling outage began.  |
| March 27, 1992<br>at 0750 EST | Snubber mark No. 2-CVCH-931 was observed to be disconnected from the pipe clamp, and various pipe support attachment bolts were loose or missing. |

D. Other Systems or Secondary Functions Affected

None.

E. Method of Discovery

The condition of snubber mark No. 2-CVCH-931 was discovered during performance of a surveillance instruction on an adjacent snubber.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

| FACILITY NAME (1)             | DOCKET NUMBER (2) | LER NUMBER (6) |          |        | PAGE (3) |
|-------------------------------|-------------------|----------------|----------|--------|----------|
|                               |                   | SEQUENTIAL     | REVISION |        |          |
|                               |                   | YEAR           | NUMBER   | NUMBER |          |
| Sequoyah Nuclear Plant Unit 1 |                   | 05             | 03       | 12     | 18       |
|                               |                   | 9              | 2        | --     | 0        |
|                               |                   | 0              | 0        | 3      | --       |
|                               |                   | 0              | 0        | 0      | 3        |
|                               |                   | 0              | 0        | 4      |          |

TEXT (If more space is required, use additional NRC Form 366A's) (17)

F. Operator Actions

No operator action was required; the unit was in Mode 6 (in process of refueling), and operability of the RCP seal leakoff piping was not required.

G. Safety System Responses

Not applicable - no safety system responses were required.

III. CAUSE OF THE EVENT

A. Immediate Causes

The support was not reassembled upon completion of related work activities.

B. Root Cause

The root cause has not yet been determined. Discrepancies that have been identified in the work document are under continuing investigation.

C. Contributing Factors

No contributing factors have been determined. Contributing factors will be determined upon completion of the investigation.

IV. ANALYSIS OF THE EVENT

The potential consequence of an inoperable snubber is an increase in the probability of structural damage to piping or components, as a result of a seismic or other event initiating dynamic loads. An engineering analysis was performed, demonstrating that the piping system (including pipe stresses, loads on other pipe supports, nozzle loads on the RCP, and valve accelerations) remained within allowable limits for this specific condition. Therefore, plant safety and operability were not adversely affected by the condition.

V. CORRECTIVE ACTIONS

A. Immediate Corrective Actions

The pipe snubber and associated support were restored to an operable condition.

A field walkdown of 16 other snubbers installed on the No. 1 seal leakoff and the No. 1 seal bypass piping at all four reactor coolant pumps was performed. The other snubbers were found properly installed.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

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|-------------------------------|-------------------|----------------|-------------------|-----------------|----------|----|----|
|                               |                   | YEAR           | SEQUENTIAL NUMBER | REVISION NUMBER |          |    |    |
| Sequoyah Nuclear Plant Unit 1 | 05000328          | 92             | 003               | 00              | 00       | 04 | 04 |

TEXT (If more space is required, use additional NRC Form 366A's) (17)

B. Corrective Action to Prevent Recurrence

The event is under investigation to determine the root cause and contributing factors and development of applicable corrective actions. After the investigation is completed, TVA will notify NRC of the investigation results by supplement to this LER.

VI. ADDITIONAL INFORMATION

A. Failed Components

None.

B. Previous Similar Events

A review of previous events identified one LER associated with this event. LER 327/88040 addressed the failure to ensure operability of a safety-related snubber during unit Modes 1 through 4. This condition was found during closure review of a work document and was attributed to scheduling inadequacies. Although the cause of the event described in this LER has not yet been determined, it is expected that the corrective action of LER 327/88040 would not have been expected to have prevented this event.

VII. COMMITMENTS

After the investigation is completed, TVA will notify NRC of the investigation results by supplement to this LER.