



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-318

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 187
License No. DPR-69

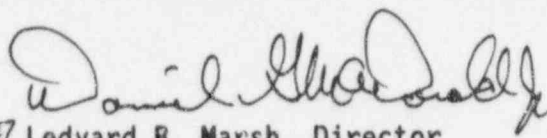
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Baltimore Gas and Electric Company (the licensee) dated October 2, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2. of Facility Operating License No. DPR-69 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 187, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. As of the date of issuance to be implemented during the Unit No. 1 spring 1996 refueling outage.

FOR THE NUCLEAR REGULATORY COMMISSION


Ledyard B. Marsh, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 19, 1995

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 187 FACILITY OPERATING LICENSE NO. DPR-69

DOCKET NO. 50-318

Revise Appendix A as follows:

Remove Pages

3/4 7-16

3/4 7-17

Insert Pages

3/4 7-16

3/4 7-17*

*Indicates rollover page

3/4.7 PLANT SYSTEMS

3/4.7.6 CONTROL ROOM EMERGENCY VENTILATION SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.6.1 The Control Room Emergency Ventilation System shall be **OPERABLE** with:

- a. Two filter trains,
- b. Two air conditioning units,
- c. Two isolation valves in each Control Room outside air intake duct,
- d. Two isolation valves in the common exhaust to atmosphere duct, and
- e. One isolation valve in the toilet area exhaust duct.

APPLICABILITY: **MODES 1, 2, 3 and 4.**

ACTION:

- a. With one filter train inoperable, restore the inoperable train to **OPERABLE** status within 7 days* or be in at least **HOT STANDBY** within the next 6 hours and in **COLD SHUTDOWN** within the following 30 hours.
- b. With one air conditioning unit inoperable, restore the inoperable unit to **OPERABLE** status within 7 days* or be in at least **HOT STANDBY** within the next 6 hours and in **COLD SHUTDOWN** within the following 30 hours.
- c. With one isolation valve per Control Room outside air intake duct inoperable, operation may continue provided the other isolation valve in the same duct is maintained closed; otherwise, be in at least **HOT STANDBY** within 6 hours and in **COLD SHUTDOWN** within the following 30 hours.
- d. With one common exhaust to atmosphere duct isolation valve inoperable, restore the inoperable valve to **OPERABLE** status within 7 days or be in at least **HOT STANDBY** within the next 6 hours and in **COLD SHUTDOWN** within the following 30 hours.

* This Action is extended from 7 days to 30 days (for loss of the emergency power supply only) during the Unit 1 1996 refueling outage. This extension begins when Emergency Diesel Generator No. 11 is removed from service on 4 kV Bus No. 11. The extension ends when Emergency Diesel Generator No. 1A is declared **OPERABLE** on 4 kV Bus No. 11 under Technical Specification 3.8.1.2 or 30 days has expired, whichever is first.

3/4.7 PLANT SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

- e. With the toilet area exhaust duct isolation valve inoperable, restore the inoperable valve to **OPERABLE** status within 24 hours or be in at least **HOT STANDBY** within the next 6 hours and in **COLD SHUTDOWN** within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.7.6.1 The Control Room Emergency Ventilation System shall be demonstrated **OPERABLE**:

- a. At least once per 62 days, on a **STAGGERED TEST BASIS**, by deenergizing the backup Control Room air conditioner and verifying that the emergency Control Room air conditioners maintain the air temperature $\leq 104^{\circ}\text{F}$ for at least 12 hours when in the recirculation mode.
- b. At least once per 31 days by initiation flow through each HEPA filter and charcoal adsorber train and verifying that each train operates for at least 15 minutes.
- c. At least once per 18 months or (1) after any structural maintenance on the HEPA filter or charcoal adsorber housing, or (2) following painting, fire or chemical release in any ventilation zone communicating with the system by:
 - 1. Verifying that the charcoal adsorbers remove $\geq 99\%$ of a halogenated hydrocarbon refrigerant test gas when they are tested in-place in accordance with Regulatory Positions C.5.a and C.5.d of Regulatory Guide 1.52, Revision 2, March 1978, while operating the ventilation system at a flow rate of $2000 \text{ cfm} \pm 10\%$.
 - 2. Verifying that the HEPA filter banks remove $\geq 99\%$ of the DOP when they are tested in-place in accordance with Regulatory Positions C.5.a and C.5.c of Regulatory Guide 1.52, Revision 2, March 1978, while operating the ventilation system at a flow rate of $2000 \text{ cfm} \pm 10\%$.
 - 3. Verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained from an adsorber tray or from an adsorber test tray in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978, demonstrates a removal efficiency of $\geq 90\%$ for radioactive methyl iodine when the sample is tested in accordance with ANSI N510-1975 (30°C , 95% R.H.).