

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Washington Nuclear Plant - Unit 2

DOCKET NUMBER (2)
0 5 0 0 0 3 9 7 1 OF 0 2

PAGE (3)
1 OF 0 2

TITLE (4)

EVENT DATE (8)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (9)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
08	01	84	84	08	0	08	24	84			0 5 0 0 0
											0 5 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 50. (Check one or more of the following) (11)

OPERATING MODE (9)	1	20.402(b)	20.406(e)	Y	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)	015	20.406(a)(1)(i)	50.38(a)(1)		50.73(a)(2)(v)	73.71(e)
		20.406(a)(1)(ii)	50.38(a)(2)		50.73(a)(2)(vi)	X OTHER (Specify in Abstract below and in Text, NRC Form 365A)
		20.406(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)	50.72(b)(2)(ii)
		20.406(a)(1)(iv)	50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)	
		20.406(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME: R. L. Koenigs, Compliance Engineer

TELEPHONE NUMBER: 509 377-2501

AREA CODE: 509

Ext. 2279

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	JNA		W120	No					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (if yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

A Turbine Stop Valve Closure Reactor Trip occurred as a result of pressurizing the turbine first stage shell above the 30% power equivalent pressure switch setpoint during a routine operator initiated turbine trip test while starting up the turbine generator.

Immediate corrective action was to recover the Plant per operating procedures, trouble-shoot and recalibrate DEH input circuitry.

No further corrective action was necessary.

8408290024 840824
PDR ADUCK 05000397
S PDR

IEE
11

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Washington Nuclear Plant - Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 9 7 8 4	LER NUMBER (8)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		- 0 8 1 0	- 0 1 0 0	1 2	OF	0 1 2

TEXT (If more space is required, use additional NRC Form 388A's) (17)

- a) Power Level - 15%
- b) Plant Mode - 1
- c) During Power Ascension Testing Program
- d) Generator Load - 0

Event

On 8/1/84 during the Power Ascension Testing Program while performing a turbine generator startup, a routine operator initiated turbine trip was accomplished. As turbine speed was stabilizing, the operator began recovery from the trip by relatching the turbine and initiating TV/GV transfer (Throttle Valve/Governor Valve control). Due to minor deviations in the throttle valve position and turbine speed DEH (Digital Electro-Hydraulic Turbine Control System) input circuit parameters and the effect of turbine speed not being completely stabilized, the control system responded by opening both the governor and throttle valves at the same time. This action allowed the turbine first stage to be pressurized to greater than 131.5 psig and actuate the pressure switches which provide input to the Reactor Protective System for the less than 30% power level bypass of the Turbine Stop Valve Closure Reactor Trip. When the 30% power bypass was removed, since turbine throttle valves were positioned less than the 90% open setpoint, a Reactor Trip occurred.

Immediate Corrective Action

The Plant was recovered per Plant Operating Procedure PPM 3.3.1.

Further Evaluation and Corrective Action

Troubleshooting by Controls Engineer revealed no failure. However, several minor calibration adjustments were analyzed as being advisable to mitigate possible additive errors in DEH input parameters. These adjustments were made and retested satisfactorily during the subsequent startup on 8/2/84.

Safety Significance

The Turbine Stop Valve Closure Reactor Trip posed no threat to the health and safety of Plant personnel or to the public because the Plant Protective System functioned as designed and because no actual protective system design parameters were exceeded.

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

August 24, 1984

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

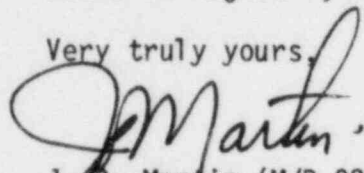
Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 84-080

Dear Sir:

Transmitted herewith is Licensee Event Report No. 84-080 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

This is the follow-up report to the verbal notification given at 2110 hours on August 1, 1984.

Very truly yours,



J. D. Martin (M/D 927M)
WNP-2 Plant Manager

JDM:mm

Enclosure:

Licensee Event Report No. 84-080

cc: Mr. John B. Martin, NRC - Region V
Mr. A. D. Toth, NRC - Site (901A)
Ms. Dottie Sherman, ANI
INPO Records Center - Atlanta, GA

LEER
1/1