

December 15, 1995

2CAN129506

U. S. Nuclear Regulatory Commission Document Control Desk Mail Station P1-137 Washington, DC 20555

Subject:

Arkansas Nuclear One - Unit 2

Docket No. 50-368 License No. NPF-6

Monthly Operating Report

Gentlemen:

The Arkansas Nuclear One - Unit 2 Monthly Operating Report for November 1995 is attached. This report is submitted in accordance with ANO-2 Technical Specification 6.9.1.6.

Very truly yours,

Dwight C. Mims

Director, Nuclear Safety

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OPERATING DATA REPORT

DOCKET NO:

50-368

DATE:

December 15, 1995

TELEPHONE:

COMPLETED BY: M. S. Whitt (501) 858-5560

OPERATING STATUS

1.	Unit Name: Arkansas Nuclear One - Unit 2					
2.	Reporting Period: November 1-30					
3.	Licensed Thermal Power (MWt): 2,815					
4.	Nameplate Rating (Gross MWe): 942.57					
5.	Design Electrical Rating (Net MWe): 912					
6.	Maximum Dependable Capacity (Gross MWe): 897					
7.	Maximum Dependable Capacity (Net MWe): 858					
8.	If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:					
9.	Power Level To Which Restricted. If Any (Net MWe): 895					
10.	Reasons For Restrictions If Any: Self imposed power restriction to ~ 98 4% power based on					

T-hot limitations and the additional 300 steam generator plugs installed during 2P95-1.

		MONTH	YR-TO-DATE	CUMULATIVE
	Hours in Reporting Period	720.0	8,016.0	137,472.0
2.	Number of Hours Reactor was			
	Critical	283.8	6,176.2	106,737.5
3.	Reactor Reserve Shutdown			
	Hours	0.0	0.0	0.0
١.	Hours Generator On-Line	48.6	5,921.0	104,554.1
5.	Unit Reserve Shutdown Hours	0.0	0.0	0.0
j	Gross Thermal Energy Generated			
	(MWH)	54,935	16,042,542	278,384,751
1.	Gross Electrical Energy			
	Generated (MWH)	6,075	5,348,232	91,717,805
3.	Net Electrical Energy	1.7		
	Generated (MWH)	-8,978	5,081,011	87,278,799
).	Unit Service Factor	6.8	73.9	76.1
).	Unit Availability Factor	6.8	73.9	76.1
	Unit Capacity Factor			
	(Using MDC Net)	-1.5	73.9	74.0
	Unit Capacity Factor			
	(Using DER Net)	-1.4	69.5	69.6
	Unit Forced Outage Rate	85.5	6.7	10.2
	Shutdowns Scheduled Over Next 6 Mor	oths (Type, Date, and	D. ation of Each):	
	Situldowns Scheduled Over Ivext 6 Mon	iths (Type, Date, and	TO WIGH OF Each).	
i.	If Shut Down At End of Report Period. Startup:	Estimated Date of		
í	Units in Test Status (Prior to Commercial	-1 (O		

	Forecast	Achieved
INITIAL CRITICALITY		12/05/78
INITIAL ELECTRICITY		12/26/78
COMMERCIAL OPERATION		03/26/80

AVEF AGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-368
UNIT: Two
DATE: December 15, 1995
COMPLETED BY: M. S. Whitt
TELEPHONE: (501) 858-5560

MONTH November 1995

DAY	AVERAGE DAILY POWER LEVEL
	(MWe-Net)

1	********	-4
2		-10
3	***************************************	-11
4	***********	-11
5		-11
6		-11
7	***************************************	-11
8	***************************************	-12
9	******************************	-10
10		-11
11	***************************************	-11
12		-12
13	*******************************	-11
14	*****************************	-15
15	*************************	-26
16	***************************	-28
17		-12
18	*************	-27
19	***************************************	-30
20	****************************	-5
21	*************	-27
22	**************	7
23	*************	-29
24	***************************************	-31
25	*******************************	-33
26	********************************	-33
27		-34
28	*************************************	-33
29	************************************	-34
30		152

AVGS: -12

INSTRUCTION

On this format, list the average daily unit power level in MWe-Net for each day in reporting month. Complete to the nearest whole megawatt.

NRC MONTHLY OPERATING REPORT OPERATING SUMMARY NOVEMBER 1995 UNIT TWO

The month began with the unit off line for refueling outage 2R11.

At 1540 hours on the thirteenth, the plant heatup was halted due to leakage in the body to bonnet seal weld for pressurizer high point vent valve 2SV-4636-1. The unit was cooled back down in order to allow repair of the seal weld. Upon completion of this repair, plant heatup was resumed at 2035 hours on the fourteenth. A second cool down was initiated at 2036 hours on the sixteenth due to leaks on pressurizer spray valve 2CV-4651 and a blank flange on the top of the pressurizer. After the leaks were repaired, heatup was again resumed at 2145 hours on the eighteenth. The reactor attained criticality at 0412 hours on the nineteenth, and the unit was placed on line at 0447 hours on the twentieth marking the end of the refueling outage.

At 1146 hours on the twentieth, the unit was taken off line to perform the main turbine overspeed trip test. Main turbine generator lockout relays 286-G2-8 and 286-G2-9 failed to reset and the test was halted at 1150 hours. It was discovered that 286-G2-8 had a design flaw that would not allow it to be reset after it had been tripped.

After de-energizing the trip signal being sent to 286-G2-8, the lockout relays were able to be reset and the main turbine overspeed trip test was continued at 1708 hours on the wentieth. The test was completed at 1900 hours and work to modify 286-G2-8 lockout relay was started. Modifications to lockout relay 286-G2-8 were completed and the unit was placed on line at 2230 hours on the twenty-first. The unit performed normal power ascension to 30% until 1710 hours on the twenty-second when the unit was taken off line due to excessive leakage past the main generator hydrogen seal. Following repairs to the seal, the unit was placed on line at 0102 hours on the thirtieth. The month ended with the unit at 30% power.

UNIT SHUTDOWNS AND POWER REDUCTIONS REPORT FOR NOVEMBER 1995

DOCKET NO. UNIT NAME DATE

50-368 ANO Unit 2 December 15, 1995

COMPLETED BY M. S. Whitt TELEPHONE

501-858-5560

NO.	DATE	<u>TYPE</u> ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR ³	LICENSEE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE	
95-06	951101	S	379.9	С	4	N/A	ZZ	772722	The unit was off line for 2R11 Refueling Outage.	
95-07	951113	F	28.9	A	4	N/A	AB	VTV	While in heatup, 2SV-4636-1 (pressurizer high point vent valve) developed a pinhole leak in the body to bonnet seal weld. The plant was cooled back down to allow the seal weld to be repaired.	
95-08	951116	F	49.2	A	4	N/A	AB	v	During the second heatup, 2CV-4651 (pressurizer spray valve) and a blank flange on the top of pressurizer developed leaks. The plant was cooled back down to allow the leaks to be repaired.	
95-09	951120	S	1.9	н	5	N/A	TA	222722	Unit taken off line to perform main turbine overspeed trip test.	
95-10	951120	F	32.8	A	5	N/A	ТВ	86	The main turbine generator lockout relays (286-G2-8 and 286-G2-9) would not reset due to a design flaw.	
95-11	951122	F	175.9	A	5	N/A	TB	SEAL	The unit was taken off line due to excessive hydrogen leakage past the main generator hydrogen seal.	
1		2				3		4		
F: For	rced	Reason:				Method: Exhib		Exhibit	t G - Instructions	
S: Sch	eduled	A - Equipment Failure (Explain)			in)	1 - Manual		for Pre	for Preparation of Data	
		B - Maintenance of Test C - Refueling D- Regulatory Restriction E - Operator Training & License Examination				4 - Continuation 5 - Load Reduction				
								Event l	Event Report (LER) File (NUREG-0161)	
					nse Examination					
			F - Administrati			9 - Other		5		
			G - Operational					Exhibit	t I - Same Source	
		- 1	H - Other (Exp!	sin)						

Reporting Period: November 1995

REFUELING INFORMATION

- 1. Name of facility: Arkansas Nuclear One Unit 2
- 2. Scheduled date for next refueling shutdown: March 21, 1997
- 3. Scheduled date for restart following refueling: May 5, 1997
- 4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10CFR Section 50.59)?

No. No

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

N/A

 Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

None planned

- 7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool:
 - a) 177
- b) 637
- 8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

present 988 increase size by 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

DATE: 1997 (Loss of full core off-load capability)