

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Browns Ferry - Units 1, 2, and 3	DOCKET NUMBER (2) 0 5 0 0 0 2 5 9	PAGE (3) 1 OF 0 2
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TITLE (4) Residual Heat Removal Service Water Pumps Declared Inoperable Rendering a 24-hour Limiting Condition for Operation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)										
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)								
0	7	20	8	4	8	4	0	2	9	0	0	0	8	1	7	8	4	Browns Ferry - Unit 2	0 5 0 0 0 2 6 0
																	Browns Ferry - Unit 3	0 5 0 0 0 2 9 6	

OPERATING MODE (8) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 1 0 0	20.402(b)	20.406(e)	50.73(a)(2)(iv)	73.71(b)						
	20.406(a)(1)(ii)	50.36(a)(1)	50.73(a)(2)(v)	73.71(e)						
	20.406(a)(1)(iii)	50.36(a)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	20.406(a)(1)(iv)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)							
	20.406(a)(1)(v)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)							
	20.406(a)(1)(vi)	50.73(a)(2)(iii)	50.73(a)(2)(ix)							

LICENSEE CONTACT FOR THIS LER (12)	
NAME David L. Smith	TELEPHONE NUMBER AREA CODE: 2 0 1 5 7 1 2 1 9 - 1 0 8 1 6 1 5

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	

SUPPLEMENTAL REPORT EXPECTED (14):			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During normal operation, a 24-hour limiting condition of operation (LCO) was entered after SI 4.5.C.3, ASME Section XI pump pressure criteria was not met for four residual heat removal service water (RHRSW) pumps. However, the technical specification flow criteria for each pump was met. The pumps were declared inoperable. The technical specification 3.5.C.6 LCO was entered because an earlier tagout of diesel generator 1A effectively rendered its associated RHRSW pumps, A1 and A2, inoperable when the four other RHRSW pumps failed their ASME Section XI pressure requirements. This LCO lasted 13 hours as SI 4.5.C.3 was subsequently performed successfully.

During the 13-hour period all required surveillance testing on redundant components was successfully performed except units 1 and 2 valves (FCV 23-57) which were not tested as required by surveillance requirement 4.5.C.1. This was caused by personnel error. The valves had been successfully tested three days prior as part of scheduled surveillance. Live-time training was conducted with all licensed personnel concerning this entire incident. No further corrective action is necessary.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Browns Ferry - Units 1, 2, and 3	DOCKET NUMBER (2) 0 5 0 0 0 2 5 9	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	0 2 9	0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

During normal operation, unit 1 was operating at 100 percent power, unit 2 at 53 percent power, and unit 3 in a refueling outage.

Four residual heat removal service water (BI) pumps (P), B-1, B-2, C-1, and D-1 were declared inoperable after failing surveillance instruction 4.5.C.3, ASME Section XI pump pressure requirements, although the flow requirements were met. This led into a 24-hour limiting condition of operation as diesel generator (EK) 1A was earlier that day tagged out for a modification and when pumps B-1, B-2, C-1, and D-1 were declared inoperable, this rendered residual heat removal service water pumps, A-1 and A-2, inoperable (Technical Specification 1.C.2 definition).

This limiting condition of operation lasted 13 hours as surveillance test 4.5.C.3 was subsequently performed successfully. All required surveillance testing for redundant components was successfully performed except units 1 and 2 valves [FCV 23-57 (V)] which were not tested to surveillance requirement 4.5.C.1. This was caused by a personnel oversight. The valves had been successfully tested three days prior as part of scheduled surveillance testing. To preclude such future incidents, live-time training was held with all licensed personnel. No further corrective action is required.

Only two of the pumps (B-1, B-2, C-1, and D-1) were tagged out at any given time for maintenance adjustments during the 13-hour period. Therefore, since the pumps all met the flow requirements, and since at least two of the four pumps in question remained "on line" with the remaining two operable pumps C-2 and D-2, the technical specification minimum number of residual heat removal service water pumps for an accident signal was met for the limiting condition of operation period. There was no safety significance due to this event.

Previous Similar Events - None

Responsible Plant Section - OP

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, Alabama 35602

August 17, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 1 - DOCKET
NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE OCCURRENCE
REPORT BFRO-50-259/84029

The enclosed report provides details concerning residual heat removal
service water pumps declared inoperable rendering a 24-hour limiting
condition for operation. This report is submitted in accordance with
10 CFR 50.73 (a)(2)(i).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

G. T. Jones

G. T. Jones
Plant Manager
Browns Ferry Nuclear Plant

Enclosure

cc (Enclosure):

Regional Administrator
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
101 Marietta Street, Suite 2900
Atlanta, GA 30303

INPO Records Center
Suite 1500
1100 Circle 75 Parkway
Atlanta, GA 30339

NRC Resident Inspector, BFN