NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Vermont Yankee	0 5 0 0 0 2 7 1	1 OF 0 2

TITLE (4)
SLC Relief Valves Low Setpoint

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 7	2 1	8 4	8 4	0 1 3	0 0	0 8	2 0	8 4		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	X 50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
James P. Pelletier, Plant Manager	AREA CODE 8 0 2 2 5 7 - 7 7 1 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
D	B R	R V	C 7 1 1	Y					
D	B R	R V	C 7 1 1	Y					

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During routine maintenance on the SLC System, relief valves SR-11-39 A and B were found to have their setpoints below the required range, as per Tech. Spec. 4.4.A.2. The most probable cause for low setpoint was attributed to the testing technique. The valves were reset, retested, and reinstalled. The test procedure used for calibration will be updated.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Vermont Yankee	0 5 0 0 0 2 7 1	8 4	— 0 1 3	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 7/21/84 during the plant's refuel outage, routine maintenance of the SLC System's relief valves SR-11-39 A and B revealed setpoints out of the allowable limits. The Tech. Spec. setpoint limits are 1400 psig-1490 psig. The valves (39 A and B) were found to have relief setpoints of 1351 and 1300 psig respectively.

The cause of the low relief valve setpoints appeared to be in the testing method. When the valves were bench tested, the valves were not properly vented. This allowed an air bubble to form against the seat. The Hydro Pump used for the test, produced pressure oscillations, due to the constant volume pump and the compressible air bubble. This oscillation made reading the pressure gauge difficult. The valves were reset, retested and reinstalled. The valves were properly vented during the retest and the results were satisfactory. The valves were Crosby Valve and Gage Company, Model JMWK.

The low relief valve setpoints are attributed to 1 of 2 factors: 1) The testing procedure did not address the venting problem or 2) the valves setpoint drifted out of the required range. The Bench Testing procedure is being changed to require that the valves be properly vented prior to pressurization.

The SLC System is designed to pump a boron neutron absorbing solution into the reactor in the event that the control rods cannot be operated to shut down the reactor. The system has the capacity to shut the reactor down from steady state, full power operation to cold shutdown.

Since the pressure at which the relief valves operated was less then the system design pressure, overpressurization of the SLC System could not have occured. Also, the pressure at which the SLC relief valves opened was at least 25 psig higher than the SLC pump design head requirement for safe plant shutdown. Since the other system components were tested and verified operating within Tech. Spec. limits, it can be stated that this condition did not impair the ability of the system to provide the reactor with full head and flow of borated water.

Based on the above, there were no adverse consequences to the public health and safety.

Similar occurrences were reported to the Commission as LER 81-31 and 80-36.

VERMONT YANKEE NUCLEAR POWER CORPORATION

P. O. BOX 157
GOVERNOR HUNT ROAD
VERNON, VERMONT 05354

August 20, 1984

VYV84-428

U. S. Nuclear Regulatory Commission
Document No. 50-271
Washington, D.C. 20555

REFERENCES: Operating License DPR-28
Docket No. 50-271
Reportable Occurrence No. LER 84-13

Dear Sirs:

As defined by 10CFR50.73, we are reporting the attached Reportable Occurrence as LER 84-13.

Very truly yours,

James P. Pelletier

James P. Pelletier
Plant Manager

RDP/cjm

cc: Regional Administrator
USNRC Office of Inspection and Enforcement
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

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