December 13, 1995

Mr. J. E. Cross
Senior Vice President
Nuclear Power Division
Duquesne Light Company
Post Office Box 4
Shippingport, Pennsylvania 15077

SUBJECT: BEAVER VALLEY PLANT PERFORMANCE REVIEW

Dear Mr. Cross:

On November 21, 1995, Region I performed a Plant Performance Review (PPR) for Beaver Valley Units 1 and 2, covering the period of June 4, 1995 to November 11, 1995. The PPR process consists of a review of inspection findings, significant events, and other information that relates to licensee performance. The purpose of this process is to assess nuclear plant performance for trends, and to plan future inspection activities at the facility. This process will be performed for all plants approximately every six months so that two PPR assessments will be performed during the approximate 18-month Systematic Assessment of Licensee Performance (SALP) cycle.

From the review, we noted that operator strengths were maintained in response to events and the follow-up of off-normal conditions. However, we are concerned about recent instances of poor safety perspective by Operations in not aggressively questioning the acceptability of having certain equipment out of service for extended periods. The most significant example of extended inoperability involved the air ejector radiation monitor, considering the safety significant role it has in identifying a steam generator tube leak before it would degrade to a rupture. Also, we have continuing concern regarding the long-standing low number of senior reactor operators (SROs), especially considering recent losses of SROs and the effect this may have on the oversight of plant operations. We remain very interested in your root cause and corrective actions to address valve mispositioning so as to maintain proper configuration control.

Maintenance performance remained at a strong level as evidenced by improved plant material condition and equipment availability. However, concerns exist in configuration control of instrumentation and controls equipment, as evidenced by the identification of instrument valves out of normal position. We note that procurement support of maintenance activities has been a problem for some time, and further improvement is needed in ensuring parts are available on a timely basis commensurate with their safety significance. We are encouraged by your current initiative to reassess your procurement functions so as to develop the necessary corrective actions. Continued good performance of engineering activities has been demonstrated in your resolution of service water system operational performance inspection concerns and your rigorous in-depth safety system functional inspection of the safety injection systems.

Plant support activities also continued to be strong except for noteworthy weaknesses in chemistry's communication of identifying tritium in the secondary side of the plant and maintenance support for fire protection equipment. With respect to safety assessment and quality verification, we note some improvement in the quality assurance oversight of control room activities and some strengthening of offsite review committee activities with more probing questions. However, we have observed that your various oversight groups have not identified issues at an early stage where line management weaknesses have appeared, as discussed above.

Overall, we perceive a reluctance, in some instances, to take in-depth corrective actions until problems escalate or repeat as with SRO staffing levels, configuration control and procurement. We are also concerned about the above instances where station operational needs due to equipment problems have not been addressed in a timely manner. In this regard, Mr. Peter W. Eselgroth of the NRC and Mr. Thomas P. Noonan of Duquesne Light Co. agreed on November 22, 1995, to holding a management meeting in January 1996 to discuss the following topics: 1) station focus on operational needs; 2) addressing site issues in their early stages; 3) mispositioned component followup actions; 4) effectiveness of oversight groups at identifying trends worthy of additional management attention and 5) a briefing on the status of your self-assessment efforts over the past year. It has been agreed that this meeting will be held at 10:00 a.m. on January 22, 1996 at the NRC Region I office.

The enclosure provides the schedule and basis for NRC inspections of your facility planned for the remainder of the current SALP period. We will inform you of any changes required to the inspection schedule.

Sincerely,

Original Signed By:
W. D. Lanning

Richard W. Cooper, II, Director
Division of Reactor Projects

Enclosure: Planned NRC Inspections at Beaver Valley
November 12, 1995 to September 28, 1996

Docket No. 50-334; 50-412

cc w/encl:
G. S. Thomas, Vice President, Nuclear Services
T. P. Noonan, Vice President, Nuclear Operations
L. R. Freeland, Manager, Nuclear Engineering Department
B. Tuite, General Manager, Nuclear Operations Unit
K. L. Ostrowski, Manager, Quality Services Unit
R. Brosi, Manager, Nuclear Safety Department
H. R. Caldwell, General Superintendent, Nuclear Operations
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OFFICE	RI/DRP		RI/DRS	E	RI/DRA		/	
NAME	LANNING		WIGGINS		KANE			
DATE	12/6/95		12/7/95		12/12/95			

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ENCLOSURE 1

Planned NRC Inspections at Beaver Valley November 12, 1995 to September 28, 1996 (Excluding Resident Inspector Reviews)

INSPECTION PROCEDURE NUMBER	TITLE	INSP TYPE	START DATE
	OPERATIONS		
71715	Sustained Control Room Operations [SRO Work Activities Safety Assessment Review] - Review impact on SRO attention to operational details caused by extra workload in response to component mispositioning events coupled with reduced SRO staffing	RI	01/29/96
40500	Effectiveness of Licensee, Controls in Identifying, Resolving, and Preventing Problems - Focus on effectiveness of line organization self assessments and independent oversight organizations	CO	06/03/96 and 06/17/96
	MAINTENANCE		
73753	Inservice Inspection (Unit 1)	CO	04/01/96
73755	Inservice Inspection Data Review and Evaluation - Review Unit 1 steam generator eddy current testing and results, due to amount of S/G tube degradation and plugging, initial use of interim plugging criteria and discovery of first circumferential indication	RI	04/22/96
73753	Inservice Inspection (Unit 2)	CO	09/23/96
	ENGINEERING		
37550	Engineering (Visit 1)	CO	12/04/95
37550	Engineering (Visit 2)	CO	04/08/96
37550	Engineering (Visit 3)	CO	08/05/96 and 08/19/96

INSPECTION PROCEDURE NUMBER	TITLE	INSP TYPE	START DATE
	PLANT SUPPORT		
84750	Radioactive Waste Treatment - Confirmatory Measurements - Indefinitely deferred by program office	CO	N/A
82302	Review of Exercise Objectives and Scenarios for Power Reactors	CO	12/26/95
86750	Solid Radwaste Management and Transportation of Radioactive Materials	CO	01/22/96
82301	Evaluation of Exercises for Power Reactors - Full Participation	CO	02/26/96
83750	Occupational Radiation Exposure - Unit 1 Outage	CO	03/25/96
64704	Fire Protection	CO	05/13/96
81700	Physical Security Program (Visit 2)	CO	05/20/96
82701	Operational Status of the Emergency Preparedness Program	CO	05/20/96
84750	Radioactive Waste Treatment - Effluent Monitoring	CO	06/10/96
N/A	Operational Safeguards Response Evaluation	N/A	07/08/96
83750	Occupational Radiation Exposure - Unit 2 Outage	CO	Sept 1996

Abbreviations:

CO - Core Inspection Requirement
RI - Regional Initiative Inspection
SI - Safety Issue Inspection
TI - Temporary Instruction