



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION II  
101 MARIETTA STREET, N.W.  
ATLANTA, GEORGIA 30303

Report No.: 50-413/84-75

Licensee: Duke Power Company  
422 South Church Street  
Charlotte, NC 28242

Docket No.: 50-413

License No.: CPPR-116

Facility Name: Catawba

Inspection Dates: June 25 - 29, 1984

Inspection at Catawba site near Rock Hill, South Carolina

Inspector: P. A. Taylor  
P. A. Taylor

7-12-84  
Date Signed

Approved by: Frank Jape  
F. Jape, Section Chief  
Engineering Branch  
Division of Reactor Safety

7/12/84  
Date Signed

SUMMARY

Areas Inspected:

This routine, unannounced inspection involved 31 inspector-hours on site in the areas of licensee action on previous inspection findings, preoperational test results evaluation for Solid State Protection System Functional Test, Engineered Safeguards (ESF) Functional Test, Integrated Leak Rate Test (ILRT), and plant tour.

Results:

No violations or deviations were identified.

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## REPORT DETAILS

### 1. Persons Contacted

#### Licensee Employees

- \*J. W. Hampton, Station Manager
- \*J. W. Cox, Superintendent of Technical Services
- \*H. D. Brandes, Design Engineer
- \*J. M. Rucci, Design Engineer
- \*P. G. LeRoy, Licensing Engineer
- \*W. F. Beaver, Performance Engineer
- R. Wolfgang, I&E Support Engineer
- A. Bhatnagar, Test Engineer

#### NRC Resident Inspectors

- \*P. H. Skinner
- \*P. K. Van Doorn

\*Attended exit interview.

### 2. Exit Interview

The inspection scope and findings were summarized on June 29, 1984, with those persons indicated in paragraph 1 above. The licensee acknowledged the inspection findings.

### 3. Licensee Action on Previous Enforcement Matters

- a. (Closed) Unresolved Item, 413/84-46-02, Inspector comments concerning the adequacy of OP/O/B/6100/13, Standby Shutdown Facility Operations. The inspector reviewed the subject procedure which was approved on May 13, 1984, and noted that the issue raised by the inspector had been incorporated into the operating procedure.
- b. (Closed) Unresolved Item 413/84-46-03, Valve 1KF122, spent fuel pool transfer-tube isolation valve. This valve is required to be open in order to provide the source of water to the suction of the (SSF) standby makeup pump. The licensee has incorporated into OP 1/A/6200/05, Spent Fuel Cooling System, the requirements to lock open 1KF122 and also perform independent verification in order to provide the necessary assurance that this valve will be open when required.

### 4. Unresolved Items

Unresolved items were not identified during this inspection.

## 5. Preoperational Test Results Evaluation

- a. Overall Completion Status of the Preoperational Test Program (70320B, 70329B)

The inspector conducted an audit of the licensee document control files to determine the completion status of those preoperational tests designated in the FSAR, Chapter 14. Preoperational tests not found on file were determined to be traceable to one of the following locations:

- Duke Power Company letter from H. B. Tucker to J. P. O'Reilly dated June 11, 1984. This letter identifies certain preoperational tests that will be outstanding for fuel loading. In addition, the justification to permit these tests to remain outstanding is provided and the mode by which these tests must be completed. The preoperational tests that are being considered to remain outstanding for fuel loading is primarily concerned with diesel generator testing following extensive inspection, ice condenser final loading, and the loading and testing of various ventilation system filter units.
  - Catawba Precritical Testing Status Document. This document identifies the remaining preoperational tests to be completed prior to fuel loading.
- b. Test Results Evaluation - TP 1/A/1600/03, Solid State Protection System Functional Test and TP 1/A/1200/03A, Engineered Safeguards Features (ESF) Functional test. (70322B, 70325B, 70326B)

The inspector reviewed the test results of the above test procedures to verify that:

- Test steps and data sheets were initialed and dated as required
- The test results met acceptance criteria
- Deficiencies identified during the test were evaluated and corrective action taken as required
- Procedure changes issued did not change the purpose of the test

(Closed) Inspector Followup Item 413/84-33-07, Installation of bistables in the SSPS for steam generator level control. The inspector reviewed the modification made to the steam generator level control in the solid state protection system. This design change provides an initiation logic of two-out-of-four, high-high steam generator level signal for feedwater isolation. The initial design called for a logic of two-out-of-three. Design change CNM 1399.60-0032 was implemented and accomplished under nuclear station work request No. 0256. The inspector verified installation of the design change.

### Integrated Leak Rate Test - Unit 1 (70323B)

As a part of the NRC routine inspection program, the inspector reviewed the final test report for the preoperational integrated leak rate test on Unit 1 which was performed during the period of January 10-18, 1984. The description of events and test results presented in this report are in agreement with the inspector's observations during the integrated leak rate test. The leak rate of 0.110752 wt% per day established for the accident calculated peak pressure condition is below the acceptance limit of 0.15 wt% per day and demonstrates an acceptable containment. A reduced pressure test was also performed. The results of the reduced pressure test were also acceptable.

Within the areas inspected, no violations or deviations were identified.

### 6. Plant Tours (71302B)

The inspector toured the control room, auxiliary building, containment and reactor building to observe work activities in progress, housekeeping, and tag controls on equipment.

Within the areas inspected, no violations or deviations were identified.