



50-461

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 11, 1995

Mr. Richard F. Phares
Director - Licensing
Clinton Power Station
Post Office Box 678
Mail Code V920
Clinton, Illinois 61727

SUBJECT: REVISED RESPONSE TO GENERIC LETTER 94-03, "INTERGRANULAR STRESS CORROSION CRACKING OF CORE SHROUDS IN BOILING WATER REACTORS" - CLINTON POWER STATION, UNIT NO. 1

Dear Mr. Phares:

Your letter of November 22, 1995 (U-602516), informed the staff of a change in commitment to perform the core shroud inspection as described in Generic Letter 94-03, "Intergranular Stress Corrosion Cracking of Core Shrouds in Boiling Water Reactors." Rather than performing this inspection during the sixth refueling outage (RF6) scheduled for October 1996, your letter states that this inspection is being deferred until the seventh refueling outage (RF7) currently scheduled for March 1998.

As described in the staff's letter to you dated February 10, 1995, the staff's position regarding the recommended schedule for inspection of the core shroud is in agreement with the BWR Vessel & Internals Project (BWRVIP) inspection guidelines found in GENE-523-113-084, "BWR Core Shroud Inspection and Evaluation Guidelines." This document lists the Clinton Power Station as a "Category A" plant. Licensees of Category A plants are advised to inspect their core shroud no later than the first outage after the plant surpasses 8 on-line years of operation. The BWRVIP document states that Category A plants have a very low likelihood of cracking. This is because Category A plants have a limited operational time that will minimize the extent of intergranular stress corrosion cracking (IGSCC), relatively low reactor coolant conductivities, and core shrouds that have been constructed with low carbon materials which have a higher resistance to IGSCC initiation.

As stated in your letter, the Clinton Power Station (CPS) will have approximately 7.63 on-line years of operation at the beginning of RF6. In addition, the core shroud material is constructed of type 304L stainless steel which has a low carbon content and the average reactor coolant conductivity during the first five operating cycles was 0.186 $\mu\text{S}/\text{cm}$. These values further supported your decision to defer the core shroud inspection to RF7.

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Richard F. Phares

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Based on the information provided to us, we concur that your decision is within the guidelines of the BWRVIP inspection guidelines and is, therefore, acceptable.

Sincerely,

ORIGINAL SIGNED BY:

Douglas V. Pickett, Project Manager
Project Directorate III-3
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-461

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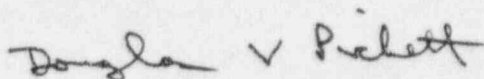
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Richard F. Phares

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Based on the information provided to us, we concur that your decision is within the guidelines of the BWRVIP inspection guidelines and is, therefore, acceptable.

Sincerely,

A handwritten signature in cursive script that reads "Douglas V. Pickett".

Douglas V. Pickett, Project Manager
Project Directorate III-3
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-461

Richard F. Phares
Clinton Power Station

Unit No. 1

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