Attachment IV to WO 95-0180 Page 1 of 3

9512190336 951213 PDR ADDCK 05000482 P PDR

.. \*

ATTACHMENT IV

PROPOSED TECHNICAL SPECIFICATION CHANGE

Attachment IV to WO 95-0180 Page 2 of 3 REACTIVITY CONTROL SYSTEMS

LIMITING CONDITION FOR OPERATION

ACTION (Continued)

- 3. The rod is declared inoperable and the SHUTDOWN MARGIN is greater than or equal to 1.3%  $\Delta k/k$ . POWER OPERATION may then continue provided that:
  - A reevaluation of each accident analysis of Table 3.1-1 is performed within 5 days; this reevaluation shall confirm that the previously analyzed results of these accidents remain valid for the duration of operation under these conditions;
  - b) A power distribution map is obtained from the movable incore detectors and  $F_Q(Z)$  and  $F^N_{\Delta H}$  are verified to be within their limits within 72 hours; and
  - c) The THERMAL POWER level is reduced to less than or equal to 75% of RATED THERMAL POWER within the next hour and within the following 4 hours the High Neutron Flux Trip Setpoint is reduced to less than or equal to 85% of RATED THERMAL POWER.
- ACTION 4 Restore the inoperable rods to OPERABLE status within 72 hours or be in HOT STANDBY within the next 6 hours.

## SURVEILLANCE REQUIREMENTS

4.1.3.1.1 The position of each full-length rod shall be determined to be within the group demand limit by verifying the individual rod positions at least once per 12 hours except during time intervals when the rod position deviation monitor is inoperable, then verify the group positions at least once per 4 hours.

4.1.3.1.2 Each full-length rod not fully inserted in the core shall be determined to be OPERABLE by movement of at least 10 steps in any one direction at least once per 31 days.

4.1.3.1.3 Prior to reactor criticality, the rod drop time of the individual full-length shutdown and control rods from the fully withdrawn position shall be demonstrated to be less than or equal to 2.7 seconds from the beginning of decay of stationary gripper coil voltage to dashpot entry, with Tavg  $\geq$  551 °F, and all reactor coolant pumps operating:

- a. For all rods following each removal of the reactor vessel head, and
- b. For specifically affected individual rods following any maintenance on or modification to the control rod drive system which could affect the drop time of those specific rods.

WOLF CREEK - UNIT 1

3/4 1-9

Amendment No. 27, 89

Attachment IV to WO 95-0180 Page 3 of 3

REACTIVITY CONTROL SYSTEMS

## BASES

## MOVABLE CONTROL ASSEMBLIES (Continued)

The ACTION statements which permit limited variations from the basic requirements are accompanied by additional restrictions which ensure that the original design criteria are met. Misalignment of a rod requires measurement of peaking factors and a restriction in THERMAL POWER. These restrictions provide assurance of fuel rod integrity during continued operation. In addition, those safety analyses affected by a misaligned rod are reevaluated to confirm that the results remain valid during future operation.

The power reduction and shutdown time limits given in ACTION statements 3.1.3.2.a.2, 3.1.3.2.b.2, and 3.1.3.2.c.2, respectively, are initiated at the time of discovery that the compensatory actions required for POWER OPERATION can no longer be met.

The maximum rod drop time restriction is consistent with the assumed rod drop time used in the safety analyses. Measurement with  $T_{avg}$  greater than or equal to 551°F and with all reactor coolant pumps operating ensures that the measured drop times will be representative of insertion times experienced during a Reactor trip at operating conditions. However, the rod drop time limit at a given RCS temperature is bounding for measurements performed at reducei temperatures because the cumulative effects of a temperature reduction result in an increase in the rod drop time. Therefore, rod drop measurement can be performed with  $T_{avg} \rightarrow T$  than 551°F provided the bounding limit of 2.7 seconds is applied.

Control rod positions and OPERABILITY of the rod position indicators are required to be verified on a nominal basis of once per 12 hours with more frequent verifications required if an automatic monitoring channel is inoperable. These verification frequencies are adequate for assuring that the applicable LCOs are satisfied.

WOLF CREEK - UNIT 1

B 3/4 1-5

Amendment No. 27, 46, 89