Omaha Public Power District
444 South 16th Street Mall
Omaha, Nebraska 68102-2247
402/636-2000

December 13, 1995
LIC-95-0233

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, D.C. 20555

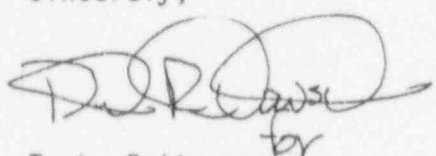
Reference: Docket No. 50-285

SUBJECT: November 1995 Monthly Operating Report (MOR)

Enclosed please find the November 1995 MOR for Fort Calhoun Station (FCS) Unit No. 1 as required by FCS Technical Specification 5.9.1.

If you should have any questions, please contact me.

Sincerely,



T. L. Patterson
Division Manager
Nuclear Operations

TLP/dll

Enclosures

c: Winston & Strawn
L. J. Callan, NRC Regional Administrator, Region IV
L. R. Wharton, NRC Project Manager
W. C. Walker, NRC Senior Resident Inspector
R. J. Simon, Westinghouse
INPO Records Center

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OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

NOVEMBER 1995
Monthly Operating Report

1. OPERATIONS SUMMARY

During the month of November, Fort Calhoun Station (FCS) operated at a nominal 100% power with the exception of a reduction to 99.3% for two days to allow a Chemical and Volume Control System Ion Exchanger to be placed in service. Normal plant maintenance, surveillance, equipment rotation activities and scheduled on-line modifications were performed during the month. Monitoring of a minor Control Element Drive Mechanism (CEDM) mechanical seal leak continued.

On November 8, 1995, a one-hour non-emergency notification was made to the NRC pursuant to 10CFR50.72(b)(1)(ii)(B) and (C) regarding the potential for the 480VAC breaker trip devices to inadvertently operate due to electrical noise in the presence of a post-accident harsh environment. Interim guidance for operating the breakers will remain in an Operations Memorandum until the devices are replaced.

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

During the month of November, no Power Operated Relief Valves (PORV) or primary system safety valve challenges or failures occurred.

3. RESULTS OF LEAK RATE TESTS

Although above normal, the November Reactor Coolant System (RCS) leak rate was steady at approximately 0.3 to 0.4 gpm throughout the month. This leak rate remained relatively steady following the reactor trip and resultant surveillance testing of the CEDMs on August 26, 1995.

The major contributor to the increase in RCS leakage has been classified as "Known" leakage. This leakage is being collected in the Reactor Coolant Drain Tank (RCDT). The leakage source has been attributed to seal leakage from CEDM #15. The "Known" leak rate stabilized at approximately 0.25 gpm. The remainder of the leakage has been classified as "Unknown" leakage.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No.	Description
None	

5. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF NOVEMBER 1995

- Replaced the module for the Reactor Coolant (RC) Loop 2A cold leg temperature channel C/TT-122C due to fluctuating DC output
- Replaced the circuit board on static inverter B to correct a bypass/transfer control problem
- Replaced a current overload relay for the blowdown tank transfer pump FW-34B
- Rebuilt the recirculation relief valve VA-287 for containment hydrogen purge fan VA-80A

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE DECEMBER 06, 1995
COMPLETED BY D. L. LIPPY
TELEPHONE (402) 533-6843

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: NOVEMBER 1995

NOTES

3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
N/A

9. Power Level to which restricted, if any (Net MWe): N/A

10. Reasons for restrictions, if any:
N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period.....	720.0	8016.0	194450.0
12. Number of Hours Reactor was Critical	720.0	6546.1	152964.0
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	720.0	6462.2	151236.5
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1077059.9	9424320.5	201573021.0
17. Gross Elec. Energy Generated (MWH)..	366805.0	3149411.0	66554293.2
18. Net Elec. Energy Generated (MWH)....	350871.3	3002871.1	63494663.4
19. Unit Service Factor.....	100.0	80.6	77.8
20. Unit Availability Factor.....	100.0	80.6	77.8
21. Unit Capacity Factor (using MDC Net)	102.0	78.4	70.6
22. Unit Capacity Factor (using DER Net)	102.0	78.4	69.0
23. Unit Forced Outage Rate.....	.0	4.1	4.0

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
A MAINTENANCE OUTAGE IS SCHEDULED TO OCCUR FROM MARCH 16-23, 1996 TO REPAIR/REPLACE DEGRADING CEDM MECHANICAL SEALS.

25. If shut down at end of report period, estimated date of startup: _____

26. Units in test status (prior to comm. oper.):

		Forecast	Achieved
INITIAL CRITICALITY			
INITIAL ELECTRICITY	N/A		
COMMERCIAL OPERATION			

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	DECEMBER 06, 1995
COMPLETED BY	D. L. LIPPY
TELEPHONE	(402) 533-6843

MONTH NOVEMBER 1995

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	487	17	487
2	486	18	488
3	486	19	488
4	487	20	487
5	488	21	487
6	488	22	487
7	487	23	487
8	485	24	487
9	486	25	488
10	487	26	488
11	488	27	488
12	488	28	488
13	488	29	488
14	487	30	488
15	488	31	N/A
16	488		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun St.
DATE December 6, 1995
COMPLETED BY D. L. Lippy
TELEPHONE (402) 533-6843

REPORT MONTH November 1995

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
NONE									

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit F - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit H - Same Source

(9/77)

Attachment IV
Refueling Information
Fort Calhoun Station - Unit No. 1

Report for the month ending November 30, 1995

1. Scheduled date for next refueling shutdown. — September 21, 1996

2. Scheduled date for restart following refueling. — November 2, 1996

3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment? — Yes

   a. If answer is yes, what, in general, will these be? — Enrichment limit of spent fuel racks is to be increased to at least 4.5 w/o from 4.2 w/o. This is necessary based upon the preliminary Cycle 17 core pattern development.

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. — N/A

   c. If no such review has taken place, when is it scheduled? — N/A

4. Scheduled date(s) for submitting proposed licensing action and support information. — January 1996 (for spent fuel rack enrichment limit change)

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. — N/A

6. The number of fuel assemblies:
   a) in the core — 133 Assemblies
   b) in the spent fuel pool — 618 Assemblies
   c) spent fuel pool storage capacity — 1083 Assemblies

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 2007 Outage

Prepared by Ken Hobbs — Date 12-6-95