



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA STREET, N.W.
ATLANTA, GEORGIA 30303

Report Nos.: 50-413/84-71 and 50-414/84-31

Licensee: Duke Power Company
422 South Church Street
Charlotte, NC 28242

Docket Nos.: 50-413 and 50-414

License Nos.: CPPR-116 and CPPR-117

Facility Name: Catawba

Inspection Date: June 12 - 15, 1984

Inspection at Catawba site near Rock Hill, South Carolina

Inspector: A. B. Ruff 7-3-84
A. B. Ruff Date Signed

Approved by: T. E. Conlon 7-3-84
T. E. Conlon, Section Chief Date Signed
Engineering Branch
Division of Reactor Safety

SUMMARY

Areas Inspected

This routine, unannounced inspection involved 23 inspector-hours on site in the areas of licensee action on previously identified enforcement matters and licensee identified items (10 CFR 50.55(e)).

Results

No violations or deviations were identified.

REPORT DETAILS

1. Persons Contacted

Licensee Employees

- *R. L. Dick, V. P. Construction
- *E. M. Couch, Project Administrator
- *L. R. Davison, Project QA Manager
- W. Henry, QA Manager
- *T. B. Bright, Construction Engineer Manager
- *G. W. Grier, Corp QA Manager
- *P. G. LeRoy, Licensing Engineer
- *W. G. Rixon, Project Control
- K. W. Schmidt, QA Engineer
- J. W. Glenn, Associate QA Engineer
- *D. P. Hensley, QA Technician
- J. Ashe, Transmission, Station Support Engineer
- C. Gadsden, Transmission, Substation Division Engineer
- W. Brown, Transmission, Substation Division Engineer
- J. Waddel, QC Department
- A. Rose, Electrical Engineer

Other licensee employees contacted included four construction craftsmen, three technicians, two mechanics, three security force members, and three office personnel.

Other Organization

M. Cooper, Rotork Controls, Inc.

NRC Resident Inspector

- *P. K. VanDoorn

2. Exit Interview

The inspection scope and findings were summarized on June 15, 1984, with those persons indicated in paragraph 1 above. The licensee acknowledged the inspection findings.

3. Licensee Action on Previous Enforcement Matters

(Open) Violation 413/84-13-01 and 414/84-09-01, Failure to Have Adequate Inspection Procedures for Protective Relaying Adjustment Activities. Duke Power Company's letter of response dated March 29, 1984, has been reviewed and determined to be acceptable by Region II. Discussions were held with Transmission, Construction and QA Department personnel and corrective actions as stated in the letter were examined. The inspector concludes that

the licensee has determined the full extent of the subject noncompliance, performed the necessary survey and followup actions to correct the present conditions, and is developing the corrective actions to preclude recurrence of similar circumstances.

Training sessions were conducted with Transmission Department personnel and the relays identified in the noncompliance have been retested by issuance of work Requests #1252 and #1253. The licensee also stated that all Unit 1 safety-related relays have been rechecked since the noncompliance was identified.

To insure that the Transmission Department's normal QC practice is controlled, such as, a two member team concurring that test data represents the settings applied to the relay being tested are properly indicated, a procedure will be issued. This procedure is to be issued in July and will have QA's concurrence. Training on this procedure will be accomplished and should be completed by mid August. This item, initially designated by NRC to be complete before fuel load, is no longer a fuel load item. However, it will remain open pending the procedure review on a future inspection.

4. Unresolved Items

Unresolved items were not identified during this inspection.

5. Licensee Identified Items (10 CFR 50.55(e))

- a. (Closed) Item 50-413, 414/83-14, Potential Deficiency of Seals at Conduit Connections to Safety-Related Equipment Located in Potential Harsh Environments. The licensee's final report was submitted on March 15, 1984. The licensee has issued several revisions of construction procedures (CP) to effect repairs in this area. Catawba Specification CNS-1390.01-00-0167 and CP-364 Rev. 2, "Sealing of Cable Entrance Fittings for Unit 1," dated April 11, 1984, is considered to meet commitments specified in the licensee's final report dated March 15, 1984. The work for Unit 1 is complete and QC has made appropriate signoffs in record copy of CP-364.

The licensee has stated that sealing prior to March 15, 1984, was in accordance with their commitments and that QC signoff in CP-364 reflects this. An in-field inspection was made of approximately 24 devices in the Dog House and UHI Accumulator Room and all were considered to be satisfactory. The licensee stated that Unit 2 sealing would be performed in accordance with Specification CNS-1390.01-00-0167 as part of the normal construction process. Unit 2 sealing will be inspected as part of NRC normal inspection program. This item is closed for both units.

- b. (Closed) Item 413, 414/84-02, "Unconsidered Effects of Superheated Steam from Steam Generators (SG) for Main Steamline Break (MSLB) Analysis." On January 19, 1984, the licensee notified Region II of the 50.55(e) item concerning the above subject. After this initial

notification, a meeting was held with the NRC staff in Bethesda, Maryland on March 19, 1984. A presentation on this subject was given to the NRC staff at that time by Westinghouse Electric Corporation and Duke Power Company. A written followup of the presentation dated March 28, 1984, was sent to the NRC. On April 4, 1984, the licensee stated that this item will continue to be followed as Confirmatory Item 12 as discussed in the Catawba Safety Evaluation Report. Based on the above and the fact that subject is covered by Confirmation Item 12, this 50.55(e) item is closed.

- c. (Closed) Item 413/84-06, Revisions to Motor Control Center (MCC) Compartments Were Not Completed. On March 29, 1984, the licensee notified Region II of the 50.55(e) item concerning the above. The final report was submitted on May 24, 1984. The report has been reviewed and determined to be acceptable by Region II. The inspector held discussions with responsible licensee representatives, and reviewed supporting documents to verify that the corrective actions identified in the report have been completed. The above deficiency occurred because changes were made to a computer printout that is identified as Catawba Nuclear Station Electrical One-Line List (CNLT). This CNLT provides information for electrical equipment such as location, overload heater size, fuse size, breaker type, etc. The CNLT changes are not always reflected on the drawings that are tabulated on the Drawing Affected List (DAL). The latter is used by construction to prepare work packages. Nonconforming Item Report (NCI) 18107 was issued by the site to track this item and shutdown (SD) requests were issued for Unit 1 to perform the necessary corrective action. The NCI and SD request for Unit 1 have been completed satisfactorily and CNLTs for MCCs and remote mounted starters are now added to the DALs.
- d. (Closed) Item 413/84-10, Defective Torque Limitors on Rotork Actuators. On April 16, 1984, the licensee notified Region II of the 50.55(e) item concerning the above. The final report was submitted on May 16, 1984. The report has been reviewed and determined to be acceptable by Region II. The inspector held discussions with licensee and contractor representatives and reviewed supporting documentation to verify that the corrective actions specified in the report have been completed. Nonconforming Item (NCI) 18185 was issued to track this item and corrective action was performed by a Rotork representative with a licensee's representative present. Corrective action was performed in accordance with Rotork procedure QC-146 and consisted of removing the locking disc and installing large set screws. The inspector observed the field change modification as it was being accomplished on one non-safety-related actuator. The field change modification has been performed for all Unit 1 safety-related actuators and NCI 18185 has been closed.
- e. (Open) Item 413/83-15, Potential Failure of Westinghouse Type SA-1 Differential Relays. On October 24, 1983, the licensee notified Region II by telephone of this 50.55(e) item and on October 31, 1983, a followup report was issued. This item was designated by NRC to be

completed before fuel load and was discussed in NRC report 84-57. The licensee indicated, as stated in NRC report 84-57, that they were investigating their installation to insure that ST Semicon Silicon-controlled rectifiers (SCRs) as identified in IEN 83-63 and their report of October 31, 1983, were not used at Catawba. This licensee has verified by work requests 1281 and 1084 that SCRs are satisfactory. A Supplemental report issued June 13, 1984, also reports this satisfactory condition. A monthly surveillance program (Work Request #3224 and #3225) has been initiated to inspect capacitors for electrolyte leakage until new replacements are installed. This item will remain open until capacitors are replaced but is no longer a fuel load item.